

March 18, 1986

Docket No. 50-334

DISTRIBUTION

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Duquesne Light Company  
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Dear Mr. Carey:

Subject: Issuance of Amendment (Licensing Action TAC 60494)

The Commission has issued the enclosed Amendment No.100 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application dated January 2, 1986.

The amendment changes the Technical Specifications for Beaver Valley Unit No. 1 to permit a one-time extension of the 12-month snubber visual inspection period. Details of this change may be found in the enclosed Safety Evaluation.

The Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/

Peter S. Tam, Project Manager  
PWR Project Directorate #2  
Division of PWR Licensing-A

Enclosures:

1. Amendment No.100 to DPR-66
2. Safety Evaluation

cc w/enclosures:  
See next page

LA:PAD#3  
CVogan  
3/ /86

PM:PAD#2  
PTam:hc *PST*  
3/14/86

D:PAD#2  
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*JER*

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3/18/86

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P PDR

Mr. J. J. Carey  
Duquesne Light Company

Beaver Valley 1 Power Station

cc:

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Duquesne Light Company

Beaver Valley 1 Power Station

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.100  
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated January 2, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.100, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This amendment is effective on issuance, to be implemented no later than 30 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director  
PWR Project Directorate #2  
Division of PWR Licensing-A

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 18, 1986

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 100 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

Remove Pages

Insert Pages

3/4 7-26

3/4 7-26

PLANT SYSTEMS

3/4.7.12 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.12 All snubbers shall be OPERABLE. The only snubbers excluded from this requirement are those installed on non-safety-related systems and then only if their failure or failure of the system on which they are installed, would have no adverse effect on any safety-related system.

APPLICABILITY: MODES 1, 2, 3 and 4. (MODES 5 and 6 for snubbers located on systems\*\* required OPERABLE in those MODES).

ACTION:

With one or more snubbers inoperable, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.12.c on the supported component or declare the supported system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.12 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

a. Visual Inspections

The first inservice visual inspection of snubbers shall be performed after four months but within 10 months of commencing POWER OPERATION and shall include all snubbers. If less than two (2) snubbers are found inoperable during the first inservice visual inspection, the second inservice visual inspection shall be performed 12 months  $\pm$  25% from the date of the first inspection. Otherwise, subsequent visual inspections shall be performed in accordance with the following schedule:

<u>No. Inoperable Snubbers per Inspection Period</u>	<u>Subsequent Visual Inspection Period* #</u>
0	18 months $\pm$ 25%
1	12 months $\pm$ 25% **
2	6 months $\pm$ 25%
3,4	124 days $\pm$ 25%
5,6,7	62 days $\pm$ 25%
8 or more	31 days $\pm$ 25%

The snubbers may be categorized into two groups: those accessible and those inaccessible during reactor operation. Each group may be inspected independently in accordance with the above schedule.

\* The inspection interval shall not be lengthened more than one step at a time.

\* The provisions of Specification 4.0.2 are not applicable.

\*\* These systems are defined as those portions or subsystems required to prevent releases in excess of 10 CFR 100 limits.

\*\* A one-time extension is granted to the above 12 month  $\pm$  25% schedule which resulted from the fourth refueling inspection activities. The visual inspections required following the fourth refueling outage will be performed during the fifth refueling outage. This extension expires upon startup from the fifth refueling outage.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTION REGULATION  
SUPPORTING AMENDMENT NO.100 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

INTRODUCTION

By a letter dated January 2, 1986, Duquesne Light Company (DLC or the licensee) requested an amendment to the Technical Specifications (Appendix A to Facility Operating License DPR-66) for Beaver Valley Power Station Unit 1. The proposed change would allow a one-time extension of the snubber visual inspection time requirements. The current snubber visual inspection is required to be completed between October 4, 1985 and April 4, 1986. This is a 12 month  $\pm$  25% interval per Technical Specification 4.7.12a due to the fact that one snubber was determined to be inoperable during the last inspection. The licensee plans to shut down the reactor on May 2, 1986 to commence the fifth refueling outage and to conduct the snubber visual inspection at that time. The reactor has not been shut down to date for a long enough period of time (2 weeks) to conduct the inspection. The reactor containment is considered inaccessible during operation (sub-atmospheric conditions and high radiation levels) for the purpose of conducting this inspection.

DISCUSSION AND EVALUATION

The technical specifications require that the snubbers installed on safety-related systems at BVPS Unit 1 be inspected on an 18-month frequency, or more often based on the results of the previous inspection; i.e. the greater the number of inoperable snubbers that are observed, then the shorter the inspection interval. During the fourth refueling outage one snubber was determined to be inoperable due to the fact that its fluid port was observed to be uncovered, therefore the visual surveillance was shortened to a 12-month  $\pm$ 25% interval. The proposed revision to the technical specification is a one-time extension of the snubber visual inspection time requirement by an additional month (from a maximum of 15 months allowed by technical specification to 16 months).

The identified problem was the result of a leak in a fluid line from the snubber remote reservoir. The snubber was function-tested satisfactorily per technical specifications after venting the snubber. The remote reservoir line was repaired and leak-checked, and all snubbers of the same design were inspected for adequate hydraulic fluid supply and found to be satisfactory.

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Based on the satisfactory testing of the problem snubber, prompt repair of the fluid line and precautionary measures taken to inspect all remote reservoir lines without further problems being identified, the staff finds that the licensee's request for a one-time one-month extension of the snubber visual inspection surveillance interval to be acceptable.

Additionally, Technical Specifications for recently licensed plants (NTOL's) allow snubbers that are observed to have the fluid port uncovered during the visual inspection to be declared operable if a satisfactory functional test is accomplished in the "as found condition".

We have evaluated the proposed change to the Technical Specifications and conclude that this change is administrative and does not involve any physical change to the plant's safety-related structures, systems, or components. Further, this change does not increase the likelihood of a malfunction of safety-related equipment, or increase the consequences of an accident previously analyzed or create the possibility of a malfunction different from those previously evaluated. Therefore, as stated above, we find the licensee's requested change to be acceptable.

#### ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such findings. Accordingly this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 18, 1986

Principal Contributor:

David M. Johnson