

April 14, 1986

Docket No. 50-334

Distribution:

Mr. J. J. Carey, Vice President
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Duquesne Light Company
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Dear Mr. Carey:

Subject: Issuance of Amendment (Licensing Action TAC 60622 and 60623)

The Commission has issued the enclosed Amendment No. 10 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in partial response to your application dated January 24, 1986.

The amendment changes the Technical Specifications for Beaver Valley Unit No. 1 to (1) add "Mode 4" to Table 4.3-2 so it is consistent with Table 3.3-3, and (2) change Surveillance Requirement 4.4.6.3 by replacing "operational condition 1" with "Mode 1" to provide consistency with current nomenclature.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/

Peter S. Tam, Project Manager
PWR Project Directorate #2
Division of PWR Licensing-A

Enclosures:

1. Amendment No. 10 to DPR-66
2. Safety Evaluation

cc w/enclosures:
See next page

LA: PAD#2
D. Miller
4/2/86

PM: PAD#2
P. Tam
3/3/86

D: PAD#2
L. Rubenstein
4/10/86

OELD
B. Barnhart
4/4/86

Mr. J. J. Carey
Duquesne Light Company

Beaver Valley 1 Power Station

cc:

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Duquesne Light Company

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Beaver Valley 1 Power Station

cc:

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Regional Administrator, Region I
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 101
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated January 24, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.101, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This amendment is effective on issuance, to be implemented no later than 30 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 14, 1986

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 101 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
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3/4 4-14b	3/4 4-14b

TABLE 4.3-2 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION
SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION	CHANNEL FUNCTIONAL TEST	MODES IN WHICH SURVEILLANCE REQUIRED
4. STEAM LINE ISOLATION				
a. Manual	N.A.	N.A.	M(1)	1, 2, 3, 4
b. Automatic Actuation Logic	N.A.	N.A.	M(2)	1, 2, 3, 4
c. Containment Pressure-- Intermediate-High-High	S	R	M	1, 2, 3
d. Steam Line Pressure--Low	S	R	M	1, 2, 3
e. Steamline Pressure Rate-High	S	R	M	1, 2, 3, 4
5. TURBINE TRIP AND FEEDWATER ISOLATION				
a. Steam Generator Water Level-High-High	S	R	M	1, 2, 3
6. LOSS OF POWER				
a. 4.16kv Emergency Bus Undervoltage (Loss of Voltage) Trip Feed & Start Diesel	N.A.	R	M	1, 2, 3, 4
b. 4.16kv and 480v Emergency Bus Undervoltage (Degraded Voltage)	N.A.	R	M	1, 2, 3, 4

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AMENDMENT NO. 40 101

REACTOR COOLANT SYSTEMS

SURVEILLANCE REQUIREMENT

- 4.4.6.3.1 Periodic leakage testing (a) on each valve listed in Table 4.4-3 shall be accomplished prior to entering Mode 1 after every time the plant is placed in the cold shutdown condition for refueling, after each time the plant is placed in a cold shutdown condition for 72 hours if testing has not been accomplished in the preceding 9 months and prior to returning the valve to service after maintenance, repair or replacement work is performed.
- 4.4.6.3.2 Whenever integrity of a pressure isolation valve listed in Table 4.4-3 cannot be demonstrated the integrity of the remaining valve in each high pressure line having a leaking valve shall be determined and recorded daily. In addition, the position of the other closed valve located in the high pressure piping shall be recorded daily.

(a) To satisfy ALARA requirements, leakage may be measured indirectly (as from the performance of pressure indicators) if accomplished in accordance with approved procedures and supported by computations showing that the method is capable of demonstrating valve compliance with the leakage criteria.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 101 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

INTRODUCTION

By letter dated January 24, 1986, the licensee proposed a number of changes to the Beaver Valley 1 Technical Specifications. The proposed Technical Specification changes would (1) add "Mode 4" to Table 4.3-2 to provide consistency with the applicable modes specified for this function in Table 3.3-3, and (2) revise Surveillance Requirement 4.4.6.3 by replacing "operational condition 1" with "Mode 1."

DISCUSSION AND EVALUATION

- (1) Table 3.3-3, Functional Unit 4.e (Steam Line Isolation, High Steam Pressure Rate) specifies Mode 4 as an applicable mode for the Engineered Safety Feature Actuation System Instrumentation System. The staff therefore agrees with the licensee that the addition of Mode 4 for Functional Unit 4.e in Table 4.3-2 pertaining to surveillance requirements adds consistency to the technical specifications and should improve the overall safety and reliability of the operation. The proposed change is acceptable.
- (2) Surveillance Requirement 4.4.6.3 currently uses an obsolete term, "operational condition 1." This term would be replaced with a current one, "Mode 1." This change is editorial and is acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). This amendment also involves changes in recordkeeping, reporting or administrative

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procedures or requirements. Accordingly, with respect to these items, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(10). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 14, 1986

Principal Contributor:

A. Toalston