

March 21, 1990

Docket Nos. 50-334  
and 50-412

Mr. J. D. Sieber, Vice President  
Nuclear Group  
Duquesne Light Company  
Post Office Box 4  
Shippingport, Pennsylvania 15077

Dear Mr. Sieber:

SUBJECT: BEAVER VALLEY UNITS 1 AND 2 - ISSUANCE OF AMENDMENT  
(TAC NOS. 75149 AND 75150)

The Commission has issued the enclosed Amendment No. 151 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit 1, and Amendment No. 28 for Facility Operating License No. NPF-73 for Unit 2, in response to your application dated October 16, 1989.

The amendments revise miscellaneous requirements in the Technical Specifications for both units as follows: testing frequency of the Auxiliary River Water System (Unit 1) and Standby Service Water System (Unit 2), testing frequency of the Supplementary Leak Collection System, sealed sources, and the Unit 1 fuel building ventilation system.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

*Original signed by*

Peter S. Tam, Senior Project Manager  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 151 to DPR-66
2. Amendment No. 28 to NPF-73
3. Safety Evaluation

cc w/enclosures:  
See next page

Concurrence page 1 of 2

OFC	:LA:PDI-4	:PM:PDI-4	:PD:PDI-4	:OGC	:	:	:
NAME	:SNOrris	:PTam:Im	:JStolz	:C. Woodhead	:	:	:
DATE	:3/6/90	:3/6/90	:3/22/90	:3/19/90	:	:	:

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Document Name: AMEND TAC 75149

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*Original signed by*  
Peter S. Tam, Senior Project Manager  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

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cc w/enclosures:

See next page

Concurrence page 2 of 2

OFC	:LA:PDI-4	:PM:PDI-4	:PD:PDI-4	:OGC *	:BC: EMEB		
NAME	:SNorris	:PTam:lm <i>PS</i>	:JStolz <i>PS Tam for</i>		: LMarsh		
DATE	:3/6/90	:3/6/90	3/22/90	: 1 / 90	: 3/13/90		

OFFICIAL RECORD COPY  
Document Name: AMEND TAC 75149

\*See previous concurrence page

Concurrence applicable to changes (1) & (2) in the safety evaluation

DATED: March 21, 1990

AMENDMENT NO. 151 TO FACILITY OPERATING LICENSE NO. DPR-66 UNIT 1  
AMENDMENT NO. 28 TO FACILITY OPERATING LICENSE NO. NPF-73 UNIT 2

DISTRIBUTION

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NRC & Local PDR

Plant File

S. Varga (14E4)

B. Boger (14A2)

J. Stolz

S. Norris

P. Tam

OGC

D. Hagan (MNBB 3302)

E. Jordan (MNBB 3302)

G. Hill(8) (P1-137)

W. Jones (P-130A)

J. Calvo (11F23)

ACRS (10)

GPA/PA

ARM/LFMB



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

March 21, 1990

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A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Peter S. Tam".

Peter S. Tam, Senior Project Manager  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 151 to DPR-66
2. Amendment No. 28 to NPF-73
3. Safety Evaluation

cc w/enclosures:  
See next page

Mr. J. Sieber  
Duquesne Light Company  
cc:

Jay E. Silberg, Esquire  
Shaw, Pittman, Potts and Trowbridge  
2300 N Street, N.W.  
Washington, DC 20037

Kenny Grada, Manager  
Nuclear Safety  
Duquesne Light Company  
P. O. Box 4  
Shippingport, Pennsylvania 15077

Ashley C. Schannauer  
Assistant City Solicitor  
City of Pittsburgh  
313 City-County Building  
Pittsburgh, Pennsylvania 15219

Commissioner Roy M. Smith  
West Virginia Department of Labor  
Building 3, Room 319  
Capitol Complex  
Charleston, WV 25305

John D. Borrows  
Director, Utilities Department  
Public Utilities Commission  
180 East Broad Street  
Columbus, Ohio 43266-0573

Director, Pennsylvania Emergency  
Management Agency  
Post Office Box 3321  
Harrisburg, Pennsylvania 17105-3321

Beaver Valley Power Station  
Units 1 & 2

Bureau of Radiation Protection  
Pennsylvania Department of  
Environmental Resources  
ATTN: R. Janati  
Post Office Box 2063  
Harrisburg, Pennsylvania 17120

Mayor of the Borough of  
Shippingport  
Post Office Box 3  
Shippingport, Pennsylvania 15077

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, Pennsylvania 19406

Resident Inspector  
U.S. Nuclear Regulatory Commission  
Post Office Box 181  
Shippingport, Pennsylvania 15077



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 151  
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated October 16, 1990 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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P FDC

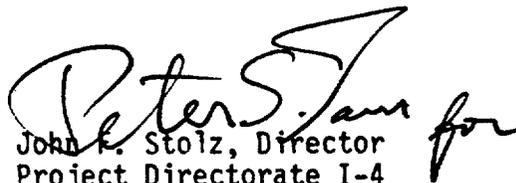
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 151, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John K. Stolz, Director  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 21, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 151

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of Appendix A (Technical Specifications) with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 7-34

3/4 7-19

3/4 7-22

3/4 7-23

B3/4 9-3

Insert

3/4 7-34

3/4 7-19

3/4 7-22

3/4 7-23

B3/4 9-3

PLANT SYSTEMS

3/4.7.13 AUXILIARY RIVER WATER SYSTEM

LIMITING CONDITION FOR OPERATION

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3.7.13.1 At least one auxiliary river water subsystem shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With less than one ARWS subsystem OPERABLE, restore at least one subsystem to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following thirty hours.

SURVEILLANCE REQUIREMENTS

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4.7.13.1 At least one ARWS subsystem shall be demonstrated OPERABLE:

- a. At least once per 92 days, by verifying that each pump develops at least 60 psig discharge pressure, while pumping through its test flow line.
- b. At least once per 18 months during shutdown by starting an Auxiliary River Water System Pump, shutting down one Reactor Plant River Water System Pump, and verifying that the Auxiliary River Water Subsystem provides at least 8000 gpm cooling water to that portion of the Reactor Plant River Water System under test for at least 2 hours.

PLANT SYSTEMS

3/4.7.8 SUPPLEMENTAL LEAK COLLECTION AND RELEASE SYSTEM (SLCRS)

LIMITING CONDITION FOR OPERATION

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3.7.8.1 Two SLCRS exhaust air filter trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With one SLCRS exhaust air filter train inoperable, restore the inoperable train to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

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4.7.8.1 Each SLCRS exhaust air filter train shall be demonstrated OPERABLE:

- a. At least once per 31 days on a STAGGERED TEST BASIS by:
  1. Initiating, from the control room, flow through the HEPA filter and charcoal adsorber train and verifying that the train operates for at least 15 minutes.
- b. At least once per 12 months or after every 720 hours of system operation or (1) after each complete or partial replacement of a HEPA filter or charcoal adsorber bank, or (2) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (3) following painting, fire or chemical release in any ventilation zone communicating with the system by:
  1. Verifying that the charcoal adsorbers remove  $\geq 99\%$  of a halogenated hydrocarbon refrigerant test gas when they are testing in-place in accordance with ANSI N510-1975 while operating the ventilation system at a flow rate of 36,000 cfm  $\pm 10\%$ .

## PLANT SYSTEMS

### 3/4.7.9 SEALED SOURCE CONTAMINATION

#### LIMITING CONDITION FOR OPERATION

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3.7.9.1 Each sealed source containing radioactive material either in excess of 100 microcuries of beta-and/or gamma-emitting material or 5 microcuries of alpha-emitting material shall be free of  $\geq 0.005$  microcuries of removable contamination.

APPLICABILITY: At all times.

#### ACTION:

- a. With a sealed source having removable contamination in excess of the above limit, immediately withdraw the sealed source from use and either:
  1. Decontaminate and repair the sealed source, or
  2. Dispose of the sealed source in accordance with Commission Regulations.
- b. The provisions of Specification 3.0.3 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.7.9.1.1 Test Requirements - Each sealed source shall be tested for leakage and/or contamination by:

- a. The licensee, or
- b. Other persons specifically authorized by the Commission or an Agreement State.

The test method shall have a detection sensitivity of at least 0.005 microcuries per test sample.

4.7.9.1.2 Test Frequencies - Each category of sealed sources (excluding startup sources and fission detectors previously subjected to core flux) shall be tested at the frequency described below.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

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- a. Sources in use - At least once per six months for all sealed sources containing radioactive materials.
  1. With a half-life greater than 30 days (excluding Hydrogen 3) and
  2. In any form other than gas.
- b. Stored sources not in use - Each sealed source and fission detector shall be tested prior to use or transfer to another licensee unless tested within the previous six months. Sealed sources and fission detectors transferred without a certificate indicating the last test date shall be tested prior to being placed into use.
- c. Startup sources and fission detectors - Each sealed startup source and fission detector shall be tested within 31 days prior to being subjected to core flux or installed in the core and following repair or maintenance to the source.

4.7.9.1.3 Reports - A Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.9.2 on an annual basis if sealed source or fission detector leakage tests reveal the presence of  $\geq 0.005$  microcuries of removable contamination.

## REFUELING OPERATIONS

### BASES

#### 3/4.9.10 AND 3/4.9.11 WATER LEVEL - REACTOR VESSEL AND STORAGE POOL

The restrictions on minimum water level ensure that sufficient water depth is available to remove 99% of the assumed 10% iodine gap activity released from the rupture of an irradiated fuel assembly. The minimum water depth is consistent with the assumptions of the accident analysis.

#### 3/4.9.12 and 3/4.9.13 FUEL BUILDING VENTILATION SYSTEM

The limitations on the storage pool ventilation system ensure that all radioactive material released from an irradiated fuel assembly will be filtered through the HEPA filters and charcoal adsorber prior to discharge to the atmosphere. The OPERABILITY of this system and the resulting iodine removal capacity are consistent with the assumptions of the accident analysis. The spent fuel pool area ventilation system is non-safety related and only recirculates air through the fuel building. The SLCRS portion of the ventilation system is safety-related and maintains a negative pressure in the fuel building. The SLCRS flow is normally exhausted to the atmosphere without filtering, however, the flow is diverted through the main filter banks by manual actuation or on a high radiation signal.

#### 3/4.9.14 FUEL STORAGE - SPENT FUEL STORAGE POOL

The requirements for fuel storage in the spent fuel pool ensure that: (1) the spent fuel pool will remain subcritical during fuel storage; and (2) a uniform boron concentration is maintained in the water volume in the spent fuel pool to provide negative reactivity for postulated accident conditions under the guidelines of ANSI 16.1-1975. The value of 0.95 or less for  $k_{eff}$  which includes all uncertainties at the 95/95 probability/confidence level is the acceptance criteria for fuel storage in the spent fuel pool.

The Action Statement applicable to fuel storage in the spent fuel pool ensures that: (1) the spent fuel pool is protected from distortion in the fuel storage pattern that could result in a critical array during the movement of fuel; and (2) the boron concentration is maintained at  $\geq 1050$  ppm (this includes a 50 ppm conservative allowance for uncertainties) during all actions involving movement of fuel in the spent fuel pool.

The Surveillance Requirements applicable to fuel storage in the spent fuel pool ensure that: (1) the fuel assemblies satisfy the analyzed U-235 enrichment limits or an analysis has been performed and it was determined that  $K_{eff}$  is  $\leq 0.95$ ; and (2) the boron concentration meets the 1050 ppm limit.

The enrichment limitations for storage of fuel in a 3 of 4 array in the spent fuel pool is based on a nominal region average enrichment with individual fuel assembly tolerance of + or - 0.05 w/o U-235.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 28  
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated October 16, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-73 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 28, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. DLCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 21, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 28

FACILITY OPERATING LICENSE NO. NPF-73

DOCKET NO. 50-412

Replace the following pages of Appendix A (Technical Specifications) with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 7-28

3/4 7-18

3/4 7-20

3/4 7-21

Insert

3/4 7-28

3/4 7-18

3/4 7-20

3/4 7-21

PLANT SYSTEMS

3/4.7.13 STANDBY SERVICE WATER SYSTEM (SWE)

LIMITING CONDITION FOR OPERATION

---

3.7.13.1 At least one standby service water subsystem shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With less than one SWE subsystem OPERABLE, restore at least one subsystem to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following thirty hours.

SURVEILLANCE REQUIREMENTS

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4.7.13.1 At least one SWE subsystem shall be demonstrated OPERABLE:

- a. At least once per 92 days, by verifying that each pump develops at least 109 psid differential pressure, while pumping through its test flow line.
- b. At least once per 18 months during shutdown by starting a Standby Service Water System Pump, shutting down one Service Water System Pump, and verifying that the Standby Service Water Subsystem provides at least 8584 gpm cooling water to that portion of the Service Water System under test for at least 2 hours.

## PLANT SYSTEMS

### 3/4.7.8 SUPPLEMENTAL LEAK COLLECTION AND RELEASE SYSTEM (SLCRS)

#### LIMITING CONDITION FOR OPERATION

---

3.7.8.1 Two SLCRS exhaust air filter trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one SLCRS exhaust air filter train inoperable, restore the inoperable train to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

---

4.7.8.1 Each SLCRS exhaust air filter train shall be demonstrated OPERABLE:

- a. At least once per 31 days by initiating, from the control room, flow through the "standby" HEPA filter and charcoal adsorber train and verifying that the train operates for at least 15 minutes with the heater controls operational.
- b. At least once per 18 months or (1) after each complete or partial replacement of a HEPA filter or charcoal adsorber bank, or (2) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (3) following painting, fire or chemical release in any ventilation zone communicating with the system by:
  1. Verifying that the charcoal adsorbers remove  $\geq 99.95\%$  of a halogenated hydrocarbon refrigerant test gas when they are tested in-place in accordance with ANSI N510-1980 while operating the ventilation system at a flow rate of 57,000 cfm  $\pm 10\%$ .
  2. Verifying that the HEPA filter banks remove  $\geq 99.95\%$  of the DOP when they are tested in-place in accordance with ANSI N510-1980 while operating the ventilation system at a flow rate of 57,000 cfm  $\pm 10\%$ .
  3. Subjecting the carbon contained in at least one test canister or at least two carbon samples removed from one of the charcoal adsorbers to a laboratory carbon sample analysis and verifying a removal efficiency of  $\geq 99\%$  for radioactive methyl iodide at an air flow velocity of 0.7 ft/sec  $\pm 20\%$  with an inlet methyl iodide concentration of 1.5 to 2.0. mg/m<sup>3</sup>,  $\geq 70\%$  relative humidity, and 30°C  $\pm \frac{1}{2}$ °C; other test conditions shall be in accordance with ANSI N510-1980. The carbon samples not obtained from test canisters shall be taken with a slotted tube sampler in accordance with ANSI N509-1980.

## PLANT SYSTEMS

### 3/4.7.9 SEALED SOURCE CONTAMINATION

#### LIMITING CONDITION FOR OPERATION

---

3.7.9.1 Each sealed source containing radioactive material either in excess of 100 microcuries of beta and/or gamma-emitting material or 5 microcuries of alpha-emitting material shall be free of  $\geq 0.005$  microcuries of removable contamination.

APPLICABILITY: AT ALL TIMES.

ACTION:

- a. With a sealed source having removable contamination in excess of the above limit, immediately withdraw the sealed source from use and either:
  1. Decontaminate and repair the sealed source, or
  2. Dispose of the sealed source in accordance with Commission Regulations.
- b. The provisions of Specification 3.0.3 are not applicable.

#### SURVEILLANCE REQUIREMENTS

---

4.7.9.1.1 Test Requirements - Each sealed source shall be tested for leakage and/or contamination by:

- a. The licensee, or
- b. Other persons specifically authorized by the Commission or an Agreement State.

The test method shall have a detection sensitivity of at least 0.005 microcuries per test sample.

4.7.9.1.2 Test Frequencies - Each category of sealed sources (excluding startup sources and fission detectors previously subjected to core flux) shall be tested at the frequency described below.

- a. Sources in use - At least once per six months for all sealed sources containing radioactive materials.
  1. With a half-life greater than 30 days (excluding Hydrogen 3) and
  2. In any form other than gas.

## PLANT SYSTEMS

### SURVEILLANCE REQUIREMENTS (CONTINUED)

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- b. Stored sources not in use - Each sealed source and fission detector shall be tested prior to use or transfer to another licensee unless tested within the previous six months. Sealed sources and fission detectors transferred without a certificate indicating the last test date shall be tested prior to being placed into use.
- c. Startup sources and fission detectors - Each sealed startup source and fission detector shall be tested within 31 days prior to being subjected to core flux or installed in the core and following repair or maintenance to the source.

4.7.9.1.3 Reports - A Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.9.2 on an annual basis if sealed source or fission detector leakage tests reveal the presence of  $\geq 0.005$  microcuries of removable contamination.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 151 TO FACILITY OPERATING LICENSE NO. DPR-66  
AMENDMENT NO. 28 TO FACILITY OPERATING LICENSE NO. NPF-73

DUQUESNE LIGHT COMPANY  
OHIO EDISON COMPANY  
PENNSYLVANIA POWER COMPANY  
THE CLEVELAND ELECTRIC ILLUMINATING COMPANY  
THE TOLEDO EDISON COMPANY

BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-334 AND 50-412

INTRODUCTION

By letter dated October 16, 1989, Duquesne Light Company (the licensee, acting as agent for the above utilities) submitted a request to change miscellaneous requirements in the Beaver Valley Power Station Technical Specifications. We have reviewed that request and our evaluation follows:

DISCUSSION AND EVALUATION

(1) Unit 1 Specification 4.7.13.1.a Regarding the Auxiliary River Water System

The auxiliary river water pumps are classified as non-nuclear safety (NNS) and as such are not required to be tested in accordance with the Inservice Testing (IST) program per ASME Section XI. This proposed change will eliminate the current requirement for testing these NNS pumps three times as often as safety-related pumps that are only required to be tested quarterly. The maintenance and operations history of these pumps does not reflect the need to retain the monthly testing frequency. We agree with the licensee's justification and find this change acceptable.

The requirement to cycle power-operated discharge valves was eliminated by Amendment No. 117 (October 27, 1987). The justification used was that all safety-related valves are cycled as part of the IST program, and need not be cycled in addition to that. The motor-operated discharge valves of this system are already tested under the IST program. Therefore, we find it acceptable to eliminate the valve cycling requirement.

The licensee also proposed to eliminate the requirement to operate each pump for at least 15 minutes during testing. A similar 15-minute test requirement has been eliminated for safety-related pumps by Amendment No. 117, on the basis that such requirement was even more stringent than that in ASME Section XI. We therefore find the elimination of the 15-minute test requirement for these NNS pumps acceptable.

(2) Unit 2 Specification 4.7.13.1.a Regarding Standby Service Water System

Similar to the Unit 1 Auxiliary River Water surveillance requirement, Unit 2 surveillance requirement 4.7.13.1.a for the Standby Service Water System has been revised to reflect the quarterly testing requirements of the other pumps in the Technical Specifications. The licensee has committed in Updated FSAR Section 9.2.1.2.4 to test these pumps periodically during unit operation, shutdown and refueling periods. These pumps are also NNS and have no role in mitigating design basis accident consequences. On such basis, these pumps do not have to be tested in accordance with the IST program per ASME Section XI. We agree with the licensee's justification and find this change acceptable.

(3) Specification 4.7.8.1.b Regarding Supplemental Leak Collection and Release System (SLCRS) Surveillance

Surveillance requirement 4.7.8.1.b has been revised by replacing "and" with "or". The "and" implies that more than one condition must be satisfied before the surveillance testing is performed. However, the intent of the surveillance requirement is to require testing after satisfying only one of the stated conditions. This is expressed by the use of "or" where only one condition need be satisfied before the required surveillance testing is performed. The use of "or" in place of "and" renders the specification more conservative regarding surveillance frequency. We find it acceptable.

(4) Specifications 3.7.9.1, 4.7.9.1.1 and 4.7.9.1.2, All Regarding Sealed Source Contamination

These specifications have been revised to conform verbatim with corresponding requirements in the Standard Technical Specification for Westinghouse Reactors (WSTS, NUREG-0452, Revision 4). All changes are editorial except the adoption of the WSTS limiting condition for operation (LCO). The revised LCO changes the range of sources controlled by this specification to sealed sources in excess of 100 microcuries of beta-and/or gamma-emitting material or 5 microcuries of alpha-emitting material. This will substantially reduce the requirements for testing and reporting inconsequential leakage from sealed sources of low activity materials. Sealed sources are not considered in the FSAR safety analyses but are administratively controlled as a routine occupational exposure concern of the NRC-approved radiological controls program in accordance with 10 CFR Part 20.

Changes to these specifications clarify them and bring them into conformance with the staff's position as expressed in the WSTS. These changes are acceptable.

(5) Specification 4.7.9.1.3, Regarding Reporting of Sealed Source Contamination

This specification is revised to conform with the reporting requirements in Specification 6.9.2. The old specification required a 90-day report on detected contamination on sealed sources, while Specifications 6.9.2 requires an annual report. The change eliminates the conflict, but has no negative effects on operational safety since the affected requirement is concerned only with the submittal frequency of a written report. This change is acceptable.

(6) Unit 1 Bases 3/4.9.12 and 3/4.9.13, Fuel Building Ventilation System

This bases section has been revised to correct an error in describing the Supplemental Leak Collection and Release System (SLCRS). The old bases stated "The SLCRS portion of the ventilation system is safety related and continuously filters the fuel building exhaust air." This was incorrect, since although the SLCRS takes suction from the fuel building to maintain a negative pressure, the normal exhaust air is not filtered through the main filter banks. However, upon a high radiation signal, the exhaust air flow is diverted through the main filter banks prior to discharge to the atmosphere. The revised section correctly states these facts to conform with the actual design and as-built hardware, and is acceptable.

ENVIRONMENTAL CONSIDERATION

This amendments change requirements with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. We have previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 21, 1990

Principal Contributor: Peter S. Tam