

December 13, 1988

Docket Nos. 50-334
and 50-412

Mr. J. D. Sieber, Vice President
Nuclear Group
Duquesne Light Company
Post Office Box 4
Shippingport, Pennsylvania 15077

Dear Mr. Sieber:

SUBJECT: BEAVER VALLEY UNITS 1 AND 2 - ISSUANCE OF AMENDMENT
(TAC NOS. 68634 AND 68635)

The Commission has issued the enclosed Amendment Nos. 133 and 9 to Facility Operating License Nos. DPR-66 and NPF-73 for the Beaver Valley Power Station, Units 1 and 2, respectively, in response to your application dated June 22, 1988.

The amendments exclude all containment isolation weight- and spring-loaded check valves not subject to Type-C testing from the lift test requirements of Specification 4.6.3.1. The valves, however, will continue to be tested in accordance with ASME Section XI, as specified in Specification 4.0.5.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance has been sent to the Office of the Federal Register for publication.

Sincerely,

Signed by

Peter S. Tam, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 133 to DPR-66
- 2. Amendment No. 9 to NPF-73
- 3. Safety Evaluation

cc w/enclosures:
See next page

LA: PDI-4
SNorris
10/26/88

PM: PDI-4
PTam: lm
10/27/88

PD: PDI-4
JStolz
10/10/88
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New design
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BGE SPLB
Craig
10/21/88

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CP

Mr. J. Sieber
Duquesne Light Company

cc:

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Beaver Valley Power Station
Units 1 & 2

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AMENDMENT NO. 133 and 9 TO FACILITY OPERATING LICENSE NO. DPR-21 and NPF-73
BEAVER VALLEY UNITS 1 AND 2

DATED: DEC 13 1993

DISTRIBUTION

Docket File

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cc: Plant Service list

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1/1



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 133
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated June 22, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 133, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective upon issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 13, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 133

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of Appendix A (Technical Specifications) with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 6-19c

3/4 6-19k

Insert

3/4 6-19c

3/4 6-19k

TABLE 3.6-1
CONTAINMENT PENETRATIONS

PENT. NO. AREA	IDENTIFICATION/ DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME*(SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME*(SEC)
32-C	Deluge System to RHR Area	1FP-800	N/A	TV-1FP-106	N/A
33-C	High Head SI to Hot Legs	(2)1SI-84	N/A	(2)MOV-1SI-869B	N/A
34-A	Spare				
35-A	Seal Injection Water RCP 1A	(10)(2)1CH-181	N/A	(2)MOV-1CH-308A	N/A
36-A	Seal Injection Water RCP 1B	(10)(2)1CH-182	N/A	(2)MOV-1CH-308B	N/A
37-A	Seal Injection Water RCP 1C	(10)(2)1CH-183	N/A	(2)MOV-1CH-308C	N/A
38-A	Containment Sump Pump Discharge	(A)TV-1DA-100A	10	(A)TV-1DA-100B	10
39-C	Steam Generator 1A Blowdown	Closed System	N/A	(2)(A)TV-1BD-100A	20
40-A	Steam Generator 1B Blowdown	Closed System	N/A	(2)(A)TV-1BD-100B	20
41-B	Steam Generator 1C Blowdown	Closed System	N/A	(2)(A)TV-1BD-100C	20
42-C	Compressed Air to Fuel Handling Equipment	1SA-15	N/A	1SA-14	N/A
43-B	Air Activity Monitor-Out	(A)TV-1CV-102-1	5	(A)TV-1CV-102	5
44-B	Air Activity Monitor-In			(A)TV-1CV-101A (A)TV-1CV-101B	5 5
45-B	Primary Grade Water to PRT	1RC-72	N/A	(A)TV-1RC-519	12
46-A	Charging Fill Header	(10)(2)1CH-170	N/A	(2)(1)FCV-1CH-160	N/A
47-B	Instrument Air	1IA-91	N/A	1IA-90	N/A

TABLE 3.6-1

CONTAINMENT PENETRATIONS

PENT. NO. AREA	IDENTIFICATION/ DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME*(SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME*(SEC)
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Emergency Containment Airlock PH-P-2

Equalization Valve
Equalization Valve

(8)(7)1VS-172

N/A

(8)(7) 1VS-171

N/A

- (A) Containment Isolation Phase A
- (B) Containment Isolation Phase B

- (1) May be opened on an intermittent basis under administrative control.
- (2) Not subject to Type C leakage tests.
- (3) May be leakage tested with water as the test fluid.
- (4) Maximum opening time.
- (5) Applicability: During CORE ALTERATIONS or movement of irradiated fuel within containment. The provisions of specification 3.0.4 are not applicable. The containment Purge Exhaust and Supply valves will be locked shut during operation in Modes 1, 2, 3 and 4.
- (6) Not subject to the requirements of specification 3/4.6.3. Listed in TABLE 3.6-1 for information only.
- (7) Tested under Type "B" testing.
- (8) Temporarily removed and penetration plugged.
- (9) Auto open on Safety Injection recirculation signal.
- (10) Not subject to the surveillance requirements of specification 3/4.6.3. Valves tested per specification 4.0.5.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 9
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company*, et al. (the licensee) dated June 22, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

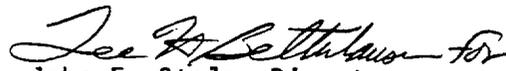
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-73 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 9, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. DLCo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective on issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 13, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 9

FACILITY OPERATING LICENSE NO. NPF-73

DOCKET NO. 50-412

Replace the following pages of the Appendix A (Technical Specifications) with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 6-17
3/4 6-18
3/4 6-19
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3/4 6-18
3/4 6-19
3/4 6-20
3/4 6-22
3/4 6-28
3/4 6-30

TABLE 3.6-1
CONTAINMENT PENETRATIONS

PENT. NO. -AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME (SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME (SEC)
1	Comp Cool from Res Heat Exch	(1)(B) 2CCP-MOV157-2 2CCP-RV105	< 60 N/A	(1)(B) 2CCP-MOV157-1	< 60
2	Comp Cool to Res Heat Exch	(1)(B) 2CCP-MOV150-2 2CCP-RV102	< 60 N/A	(1)(B) 2CCP-MOV150-1	< 60
4	Comp Cool to Res Heat Exch	(1)(B) 2CCP-MOV151-2 2CCP-RV103	< 60 N/A	(1)(B) 2CCP-MOV151-1	< 60
5	Comp Cool from Res Heat Exch	(1)(B) 2CCP-MOV156-2 2CCP-RV104	< 60 N/A	(1)(B) 2CCP-MOV156-1	< 60
6	SPARE				
7	High Head Safety Injection	(3)(2) 2SIS-83 (13)	N/A	(3)(2) 2SIS-MOV869A	N/A
9	SPARE				
11	Instrument Air	(A) 2IAC-MOV133	< 60	(A) 2IAC-MOV134	< 60
13	SPARE				
14	Chill & Service Wtr to Cont. Air Recirc Cooling coils	(B) 2SWS-MOV153-2	< 60	(B) 2SWS-MOV153-1 2SWS-RV153	< 60 N/A
15	CHARGING	(3)(2) 2CHS-31 (13)	N/A	(3)(2) 2CHS-MOV289	< 10
16	SPARE				

TABLE 3.6-1 (Cont)

CONTAINMENT PENETRATIONS

BEAVER VALLEY - UNIT 2

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Amendment No. 9

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE		MAXIMUM STROKE TIME (SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME (SEC)
17	High Head Safety Injection	(3)(2) 2SIS-84 (13)		N/A	(3)(2) 2SIS-MOV869B	N/A
19	Seal Water from Reactor Coolant Pump	(A)	2CHS-MOV378 2CHS-473	< 60 N/A	(A) 2CHS-MOV381	< 60
20	Safety Injection Accumulator Makeup		2SIS-42	N/A	(1) 2SIS-41 2SIS-RV130	N/A N/A
21	Chill & Service Wtr from Cont. Air Recirc Cooling Coils	(B)	2SWS-MOV155-2	< 60	(B) 2SWS-MOV155-1 2SWS-RV155	< 60 N/A
22	SPARE					
23	SPARE					
24	Residual Heat Removal to Refueling Water Tank		2RHS-107	N/A	2RHS-15 2RHS-RV100	N/A N/A
25	Chill & Service Wtr from Cont. Air Recirc Cooling Coils	(B)	2SWS-MOV154-2	< 60	(B) 2SWS-MOV154-1 2SWS-RV154	< 60 N/A
27	Chill & Service Wtr to Cont. Air Recirc Cooling Coils	(B)	2SWS-MOV152-2	< 60	(B) 2SWS-MOV152-1 2SWS-RV152	< 60 N/A
28	Reactor Coolant Letdown	(A) (A) (A) (1)	2CHS-AOV200A 2CHS-AOV200B 2CHS-AOV200C 2CHS-HCV142 2CHS-RV203	10 10 10 N/A N/A	(A) 2CHS-AOV204	< 60

TABLE 3.6-1 (Cont)

CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE		MAXIMUM STROKE TIME (SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME (SEC)
29	Pri Dr. Trans Pump Disch	(A)	2DGS-AOV108A	< 60	(A) 2DGS-AOV108B 2DGS-RV115	< 60 N/A
30	SPARE					
31	SPARE					
32	SPARE					
33	SPARE					
34	High Head Injection Line	(3)(2) (13)	2SIS-94	N/A	(3)(2) 2SIS-MOV836 (3)(2) 2SIS-MOV840	N/A N/A
35	Inj Seal Wtr to Reactor Coolant Pump	(3)(2) (13)	2CHS-474	N/A	(2)(3) 2CHS-MOV308A	N/A
36	Inj Seal Wtr to Reactor Coolant Pump	(3)(2) (13)	2CHS-476	N/A	(2)(3) 2CHS-MOV308B	N/A
37	Inj Seal Wtr to Reactor Coolant Pump	(3)(2) (13)	2CHS-475	N/A	(2)(3) 2CHS-MOV308C	N/A
38	Sump Pump Discharge	(A)	2DAS-AOV100A	< 60	(A) 2DAS-AOV100B 2DAS-RV110	< 60 N/A
39	St Gen Blowdown		Closed System	N/A	(2) 2BDG-AOV100A-1	< 60
40	St Gen Blowdown		Closed System	N/A	(2) 2BDG-AOV100B-1	< 60
41	St Gen Blowdown		Closed System	N/A	(2) 2BDG-AOV100C-1	< 60

BEAVER VALLEY - UNIT 2

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Amendment No. 9

TABLE 3.6-1 (Cont)

CONTAINMENT PENETRATIONS

PENT. NO. -AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME (SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME (SEC)
42	Service Air	2SAS-15	N/A	2SAS-14	N/A
43	Air Monitor Sample	2CVS-93	N/A	(A) 2CVS-SOV102	< 60
44	Air Monitor Sample	(1)(A) 2CVS-SOV153B	< 60	(1)(A) 2CVS-SOV153A	< 60
45	Primary Grade Water	2RCS-72	N/A	(A) 2RCS-AOV519 2RCS-RV100	< 60 N/A
46	Loop Fill	(3)(2) (13) 2CHS-472	N/A	(3)(2)(1) 2CHS-FCV 160	N/A
47	SPARE				
48	Primary Vent Header	(A) 2VRS-AOV109A-2	< 60	(A) 2VRS-AOV109A-1	< 60
49	Nitrogen Supply Manifold	2RCS-68	N/A	(A) 2RCS-AOV101	< 60
50	SPARE				
51	SPARE				
52	SPARE				
53	Nitrogen Manifold	(A) 2GNS-AOV101-2	< 10	(A) 2GNS-AOV101-1	< 60

BEAVER VALLEY - UNIT 2

3/4 6-20

Amendment No. 9

TABLE 3.6-1 (Cont)
CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE		MAXIMUM STROKE TIME (SEC)	OUTSIDE VALVE		MAXIMUM STROKE TIME (SEC)
60	Low Head Safety Injection Discharge	(3)(2) (13)	2SIS-132	N/A	(3)(2)	2SIS-MOV8888B	N/A
61	Low Head Safety Injection Discharge	(3)(2) (13)	2SIS-130	N/A	(3)(2)	2SIS-MOV8889	N/A
62	Low Head Safety Injection Discharge	(3)(2) (13)	2SIS-133	N/A	(3)(2)	2SIS-MOV8888A	N/A
63	Quench Pump Discharge		2QSS-4	N/A	(B)	2QSS-MOV101A 2QSS-RV101A	< 60 (4) N/A
64	Quench Pump Discharge		2QSS-3	N/A	(B)	2QSS-MOV101B 2QSS-RV101B	< 60 (4) N/A
65	Fuel Transfer Tube	(7)	Flange	N/A			
66	Recirc Spray Pump Suction				(B)(2)	2RSS-MOV155A	< 60 (4)
67	Recirc Spray Pump Suction				(B)(2)	2RSS-MOV155C	< 60 (4)
68	Recirc Spray Pump Suction				(B)(2)	2RSS-MOV155D	< 60 (4)
69	Recirc Spray Pump Suction				(B)(2)	2RSS-MOV155B	< 60 (4)
70	Recirculation Pump Discharge	(2)(13)	2RSS-29	N/A	(B)(2) (6)	2RSS-MOV156A 2RSS-RV156A	< 60 (4) N/A
71	Recirculation Pump Discharge	(2)(13)	2RSS-31	N/A	(10)(B)(2) (6)	2RSS-MOV156C 2RSS-RV156C	< 60 (4) N/A

TABLE 3.6-1 (Cont)

CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME (SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME (SEC)
105	Leak Detection			(2)	2LMS-SOV951 < 60 (4)
	Leak Detection				2LMS-51 N/A 2LMS-52 N/A
	Hydrogen Analyzer	(1)	2HCS-SOV133A N/A	(1)	2HCS-SOV134A N/A
	Post Accident Sampling	(A)(1)	2PAS-SOV105A-1 < 60	(A)(1)	2PAS-SOV105A-2 < 60
106	Safety Inj. Test Line	(A)	2SIS-MOV842 < 60	(A)	2SIS-AOV889 < 60 2SIS-RV175 N/A
108	SPARE				
110	SPARE				
113	Safety Injection	(3)(2) (13)	2SIS-95 N/A	(3)(2)	2SIS-MOV867C < 10 (4)
				(3)(2)	2SIS-MOV867D < 10 (4)
114	Recirculation Pump Discharge	(2)(13)	2RSS-32 N/A	(10)(B)(2) (6)	2RSS-MOV156D < 60 (4) 2RSS-RV156D N/A
				(B)(2) (6)	2RSS-MOV156B < 60 (4) 2RSS-RV156B N/A
115	Recirculation Pump Discharge	(2)(13)	2RSS-30 N/A	(B)(2) (6)	2RSS-MOV156B < 60 (4) 2RSS-RV156B N/A
116	Fire Protection HVR Filter B		2FPW-388 N/A	(A)	2FPW-AOV221 < 60
117	Fire Protection HVR Filter A		2FPW-382 N/A	(A)	2FPW-AOV204 < 60

BEAVER VALLEY - UNIT 2

3/4 6-28

Amendment No. 9

TABLE 3.6-1 (Cont)

NOTES:

- (A) Containment Isolation Phase A.
- (B) Containment Isolation Phase B.
- (1) May be opened on an intermittent basis under administrative control.
- (2) Not subject to Type C leakage tests.
- (3) May be leakage tested with water as the test fluid.
- (4) Maximum opening time.
- (5) Applicability: During CORE ALTERATIONS or movement of irradiated fuel within containment. The provisions of Specification 3.0.4 are not applicable. The containment Purge Exhaust and Supply valves will be locked shut during operation in modes 1, 2, 3, and 4.
- (6) Not subject to the requirements of Specification 3/4.6.3. Listed in Table 3.6-1 for information only.
- (7) Tested under Type "B" testing.
- (8) Temporarily removed and penetration plugged.
- (9) Auto open on Safety Injection recirculation signal.
- (10) Auto close on Safety Injection recirculation signal.
- (11) Auto open on QSS switchover signal.
- (12) Isolation is provided by bellows operated hydraulic isolators.
- (13) Not subject to the surveillance requirements of specification 3/4.6.3. Valves tested per specification 4.0.5.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 133 TO FACILITY OPERATING LICENSE NO. DPR-66 AND
AMENDMENT NO. 9 TO FACILITY OPERATING LICENSE NO. NPF-73

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-334 AND 50-412

INTRODUCTION

On March 22, 1983, we issued Amendment No. 65 (TAC 48717) to the Beaver Valley Unit 1 license, revising the Technical Specifications to waive the need of Type-C leakage testing for certain valves. In the Safety Evaluation associated with Amendment No. 65, we concluded that the lines associated with those valves do not represent potential containment atmospheric leakage paths.

By letter dated June 22, 1988, Duquesne Light Company (the licensee, acting as agent for the above-listed utilities) requested amendments for both units to exclude all containment isolation weight- and spring-loaded check valves not subject to Type-C leakage testing from the surveillance requirements of Specification 4.6.3.1.

DISCUSSION AND EVALUATION

(1) Table 3.6-1 (Unit 1)

The basis for surveillance requirements 4.6.3.1.1.a.2 and 4.6.3.1.2.e is to ensure that applicable containment isolation check valves will remain closed, preventing the loss of subatmospheric pressure in the containment following a design-basis accident (DBA). The Unit 1 FSAR discusses the design basis of these valves in Section 5.3.3. It states that these check valves are designed to require, in order to open, a differential pressure across the valve in the normal flow direction exceeding the expected post-DBA differential pressure between atmosphere and containment (about 1.2 psi). As a result, leakage into the containment through incoming lines with check valves inside the containment (caused by passive failures of such lines between the containment penetration and the outside isolation valve, or by failure of the outside isolation valve to close) is prevented.

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DPR ADDCK 05000334
PDC

Amendment No. 65 to the Unit 1 Technical Specifications deleted the containment Type-C leakage testing requirement for 38 containment isolation valves including the following check valves:

1CH-170
1CH-181
1CH-182
1CH-183

Approval of this change was based on the conclusion that the lines associated with these valves do not represent potential containment atmospheric leakage paths and, therefore, are not subject to the provisions of 10 CFR Part 50, Appendix J, Section III.C. This conclusion was based on the fact that these lines are expected to be operating after a DBA, and will be pressurized to well in excess of the peak containment pressure, even in the event of any single failure. The licensee has demonstrated that these lines would remain filled with water for the 30-day period following the accident. In the Safety Evaluation associated with Amendment No. 65, we found the licensee's justification acceptable.

The Unit 1 Updated FSAR, Appendix B, Table B.1-1, "Structures and Systems Requiring Design for Seismic Loading," states that all containment isolation valves and associated piping are seismically qualified. Also, per FSAR Section 5.3.1, the containment isolation system components (piping, valves, penetrations, etc.) are protected from internally or externally generated missiles and water jets.

Thus, the lines associated with these valves are not potential containment atmospheric leakage paths, and the lines are protected from failures due to seismic and missile events. The penetrations these lines make at the containment do not present a potential for loss of containment integrity following a DBA. Full-stroke tests and valve leakage tests with water would continue to be performed where required by the ASME Section XI Inservice Testing Program to assure valve operability. This change will therefore not affect the capability to isolate the containment, nor increase the potential for any release of the containment atmosphere following any postulated accidents. Therefore, the changes are acceptable.

(2) Table 3.6-1 (Unit 2)

The Unit 2 FSAR discusses containment isolation weight-loaded check valves in Section 6.2.4.1. The design basis stated for these valves is to ensure that the valves remain closed when the inside containment atmosphere returns to subatmospheric conditions following a DBA.

As with Unit 1, Unit 2 has excluded certain valves from Type-C leakage testing. Unit 2 containment isolation valves not subject to Type-C leakage testing are listed in Table 6.2-60 of the FSAR. Unit 1 Amendment No. 65 was used as the basis for justifying the exclusion of these valves (this is discussed in the Unit 2 Safety Evaluation Report dated October 1985, Page 6-18). These valves are excluded from Type-C leakage testing since they do not present potential containment atmospheric leakage paths. This conclusion is based on the fact that the lines associated with these penetrations are pressurized well in excess of the containment pressure following a DBA and will be filled with water for at least 30 days following the accident.

All containment isolation valves and the piping between these valves are seismically qualified as stated in Table 3.2-1 and Section 6.2.4.1 of the Updated FSAR. In addition, Section 6.2.4.1 states that all containment isolation fluid system components have been evaluated for the effects of postulated missiles.

Thus, we find it acceptable, based on reasons identical to those for Unit 1 above, to exclude the valves listed in Table 1 of the licensee's submittal from the lift check tests required by Specifications 4.6.3.1.1.a and 4.6.3.1.2.e. Again, as in Unit 1, full-stroke tests and valve leakage tests with water will continue to be performed where required by ASME Section XI to demonstrate valve operability.

ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an environmental assessment and finding of no significant impact have been prepared and published in the Federal Register on December 13, 1988 (53 FR 50141). Accordingly, based upon the environmental assessment, we have determined that the issuance of these amendments will not have a significant effect on the quality of the human environment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 13, 1988

Principal Contributor: Peter S. Tam

UNITED STATES NUCLEAR REGULATORY COMMISSIONDUQUESNE LIGHT COMPANYOHIO EDISON COMPANYTHE CLEVELAND ELECTRIC ILLUMINATING COMPANYTHE TOLEDO EDISON COMPANYPENNSYLVANIA POWER COMPANYDOCKET NOS. 50-334 AND 50-412NOTICE OF ISSUANCE OF AMENDMENTS TOFACILITY OPERATING LICENSES

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment Nos. 133 and 9 to Facility Operating License Nos. DPR-66 and NPF-73, respectively, issued to Duquesne Light Company, et al. (the licensee). The amendments revise the Technical Specifications for operation of the Beaver Valley Power Station, Units 1 and 2, located in Shippingport, Pennsylvania. The amendments are effective as of the date of issuance, to be implemented within 60 days of issuance.

The amendment excludes all containment isolation weight- and spring-loaded check valves not subject to Type-C testing from the lift test requirements of Specification 4.6.3.1. The valves, however, will continue to be tested in accordance with ASME Section XI, as specified in Specification 4.0.5.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments.

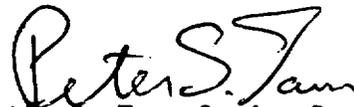
Notice of Consideration of Issuance of Amendments and Opportunity for Prior Hearing in connection with this action was published in the FEDERAL REGISTER on August 16, 1988 (53 FR 30880). No request for a hearing or petition for leave to intervene was filed following this notice.

The staff has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the staff has concluded that the issuance of these amendments will not have a significant effect on the quality of the human environment.

For further details with respect to the action, see (1) the application for amendments dated June 22, 1988, (2) Amendment Nos. 133 and 9 to License Nos. DPR-66 and NPF-73, (3) the staff's related Safety Evaluation and, (4) the staff's Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, N.W., Washington, D.C., and the B. F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania 15001. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Projects I/II.

Dated at Rockville, Maryland this 13th day of December, 1988.

FOR THE NUCLEAR REGULATORY COMMISSION



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