

December 19, 1988

Docket Nos. 50-334
and 50-412

Mr. J. D. Sieber, Vice President
Nuclear Group
Duquesne Light Company
Post Office Box 4
Shippingport, Pennsylvania 15077

Dear Mr. Sieber:

SUBJECT: BEAVER VALLEY UNITS 1 AND 2 - ISSUANCE OF AMENDMENT
(TAC NOS. 67012 AND 67013)

The Commission has issued the enclosed Amendment No. 134 to Facility
Operating License No. DPR-66 (Unit 1) and Amendment No. 11 to Facility
Operating License NPF-73 (Unit 2).

The amendments clarify several action statements regarding the reactor trip
breakers in Table 3.3-1 of the Technical Specifications of each unit, in
response to your application dated January 27, 1988 and revision dated July 26,
1988.

A copy of the related Safety Evaluation is also enclosed. The Notice of
Issuance will be included in the Commission's bi-weekly Federal Register notice.
Sincerely,

Signed by

Peter S. Tam, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 134 to DPR-66
- 2. Amendment No. 11 to NPF-73
- 3. Safety Evaluation

Concurrence page 1 of 2

cc w/enclosures:
See next page

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TBarnhart(8)

Wanda Jones
EButcher, 11/F/23
ACRS (10)
GPA/PA
ARM/LFMB

*OGC comment
incorporated into SE
PST 12/9/88*

[5520 Document Name: AMEND TACS 67012/67013

LA:PDI-4
SNorris
11/29/88

PM:PDI-4
PTam:lm
11/29/88

PD:PDI-4
JStolz
12-7/88

*OGC
K. Beckmann
12/6/88
w/ change to p. 2, SE*

*DFD
1/1
[Signature]*

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Peter S. Tam, Senior Project Manager
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LA: PDI-4 SNorris 11/29/88	PM: PDI-4 PTam: 1m 11/29/88	PD: PDI-4 JStolz 12/7/88
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OGC comment incorporated into SE
RST 12/9/88

OGC
R. Bechmann
12/6/88
no change to p. 2, SE

BC: S128
SNewberry
12/7/88

Mr. J. Sieber
Duquesne Light Company

Beaver Valley Power Station
Units 1 & 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 134
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated January 27, 1988 and revision dated July 26, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 134, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective on issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 19, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 134

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of Appendix A, Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 3-4

3/4 3-7

Insert

3/4 3-4

3/4 3-7

TABLE 3.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
18. Turbine Trip (Above P-9)					
A. Auto Stop Oil Pressure	3	2	2	1	7
B. Turbine Stop Valve Closure	4	4	4	1	8
19. Safety Injection Input from ESF	2	1	2	1,2	1
20. Reactor Coolant Pump Breaker Position Trip (Above P-7)	1/breaker	2	1/breaker per oper- ating loop	1	11
21. Reactor Trip Breakers	2 2	1 1	2 2	1,2 3*,4*,5*	40 39
22. Automatic Trip Logic	2 2	1 1	2 2	1,2 3*,4*,5*	1 39
23. Reactor Trip System Interlocks					
A. Intermediate Range Neutron Flux, P-6	2	1	1	2	3
B. Power Range Neutron Flux, P-8	4	2	3	1	12
C. Power Range Neutron Flux, P-9	4	2	3	1	12
D. Power Range Neutron Flux, P-10	4	2	3	1	12
E. Turbine Impulse Chamber Pressure, P-13	2	1	1	1	12

BEAVER VALLEY - UNIT 1

3/4 3-4

Amendment No. 88, 107, 134

TABLE 3.3-1 (CONTINUED)

- ACTION 9 With a channel associated with an operating loop inoperable, restore the inoperable channel to OPERABLE status within 2 hours or be in HOT STANDBY with the next 6 hours; however, one channel associated with an operating loop may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.1.1.
- ACTION 10 Not applicable.
- ACTION 11 With less than the Minimum Number of Channels OPERABLE, operation may continue provided the inoperable channel is placed in the tripped condition within 1 hour.
- ACTION 12 With the number of channels OPERABLE one less than required by the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or be in HOT STANDBY within the next 6 hours and/or open the reactor trip breakers.
- ACTION 39 With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or open the reactor trip breakers within the next hour.
- ACTION 40 a. With one of the diverse trip features (undervoltage or shunt trip attachment) of a reactor trip breaker inoperable, restore the diverse trip feature to OPERABLE status within 48 hours or declare the breaker inoperable and be in HOT STANDBY within 6 hours. Neither breaker shall be bypassed while one of the diverse trip features is inoperable except for the time required for performing maintenance to restore the breaker to OPERABLE status.
- b. With one reactor trip breaker inoperable as a result of something other than an inoperable diverse trip feature; be in at least HOT STANDBY within 6 hours, however, one channel may be bypassed for up to 2 hours for surveillance testing per specification 4.3.1.1, provided the other channel is OPERABLE.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 11
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated January 27, 1988 and revision dated July 26, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-73 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 11, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. DLCo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective on issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 19, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 11

FACILITY OPERATING LICENSE NO. NPF-73

DOCKET NO. 50-412

Replace the following pages of the Appendix A (Technical Specifications) with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 3-4

3/4 3-7

Insert

3/4 3-4

3/4 3-7

TABLE 3.3-1 (Continued)
REACTOR TRIP SYSTEM INSTRUMENTATION

FUNCTIONAL UNIT	TOTAL NO. OF CHANNELS	CHANNELS TO TRIP	MINIMUM CHANNELS OPERABLE	APPLICABLE MODES	ACTION
19. Safety Injection Input from ESF	2	1	2	1, 2	1
20. Reactor Coolant Pump Breaker Position Trip (Above P-7)	1/breaker	2	1/breaker per operating loop	1	11
21. Reactor Trip Breakers	2	1	2	1, 2, 3*, 4*, and 5*	40
	2	1	2		39
22. Automatic Trip Logic	2	1	2	1, 2, 3*, 4*, and 5*	1
	2	1	2		39
23. Reactor Trip System Interlocks					
a. Intermediate Range Neutron Flux, P-6	2	1	2	2	44
b. Power Range Neutron Flux, P-8	4	2	3	1	44
c. Power Range Neutron Flux, P-9	4	2	3	1	44
d. Power Range Neutron Flux, P-10	4	2	3	1, 2	44
e. Turbine Impulse Chamber Pressure, P-13	2	1	2	1	44

TABLE 3.3-1 (Continued)

- ACTION 10 - This Action is not used.
- ACTION 11 - With less than the Minimum Number of Channels OPERABLE, operation may continue provided the inoperable channel is placed in the tripped condition within 1 hour.
- ACTION 12 - With the number of channels OPERABLE one less than required by the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or be in HOT STANDBY within the next 6 hours and/or open the reactor trip breakers.
- ACTION 39 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or open the reactor trip breakers within the next hour.
- ACTION 40 -
- a. With one of the diverse trip features (undervoltage or shunt trip attachment) of a reactor trip breaker inoperable, restore the diverse trip feature to OPERABLE status within 48 hours or declare the breaker inoperable and be in HOT STANDBY within the next 6 hours. Neither breaker shall be bypassed while one of the diverse trip features is inoperable except for the time required for performing maintenance to restore the breaker to OPERABLE status.
 - b. With one reactor trip breaker inoperable as a result of something other than an inoperable diverse trip feature, be in at least HOT STANDBY within 6 hours; however, one channel may be bypassed for up to 2 hours for surveillance testing per specification 4.3.1.1, provided the other channel is OPERABLE.
- ACTION 44 - With less than the Minimum Number of channels OPERABLE, within 1 hour determine by observation of the associated permissive annunciator window(s) that the interlock is in its required state for the existing plant condition, or apply Specification 3.0.3.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 134 TO FACILITY OPERATING LICENSE NO. DPR-66
AND AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NO. NPF-73

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

BEAVER VALLEY POWER STATION, UNITS 1 AND 2

DOCKET NOS. 50-334 AND 50-412

INTRODUCTION

On February 4, 1987, we issued Amendment No. 107 to the license for Beaver Valley Unit 1, thus imposing certain requirements on testing of the reactor trip breakers, undervoltage and shunt trip attachments, and bypass breakers. Amendment No. 107 was issued as the licensee requested in its letter dated October 27, 1986. When we prepared the Technical Specifications for Unit 2, we adopted Unit 1 Amendment No. 107 in its entirety.

By letter dated January 27, 1988, Duquesne Light Company (the licensee, acting as agent for the above utilities) requested that the subject specifications be modified. We reviewed that submittal and found that the proposed changes actually added more confusion. We stated our conclusion to the licensee in telephone conversations. As a result, on July 26, 1988, the licensee submitted a revised request. Our review of the second submittal is described below.

DISCUSSION AND EVALUATION

The proposed changes for both units all pertain to Table 3.3-1, "Reactor Trip System Instrumentation." The proposed changes clarify the actions an operator is to take when a reactor trip breaker is inoperable. If a diverse trip feature is inoperable in Modes 1 or 2, then up to 48 hours are allowed to restore the breaker to operable status. If the breaker is inoperable due to other than the inoperability of a diverse trip feature, then the plant must be placed in hot standby within the next 6 hours. These changes are consistent with the guidance of Generic Letter 85-09 which describes the use of Action 40 as follows:

"With the addition of the automatic actuation of the shunt trip attachment, diverse features exist to effect a reactor trip for each breaker. If one of these diverse trip features is inoperable, a decision would have to be made with regard to the operability status of the reactor trip breaker. The definition of OPERABLE-OPERABILITY in Section 1.0 of the STS states that a component shall be operable or have operability when it is capable of performing its safety function. Since either trip feature being operable would initiate a reactor trip on demand, it would be overly conservative to treat a breaker as inoperable if one of these diverse trip features were inoperable. However, on the other hand, the reliability of the reactor trip system would be reduced if each diverse trip feature is not maintained in an operable status.

The reactor trip breaker surveillance test should independently verify the operability of the shunt and undervoltage trip features of the reactor trip breakers as part of a single sequential test procedure. Therefore, the surveillance test which identifies a failure of one diverse trip feature also confirms the operability of the other trip feature. As a consequence, there is a high degree of confidence that the operable trip feature would be capable of initiating a reactor trip in the next 48 hours."

Amendment No. 107 to the Unit 1 Technical Specifications basically stated the same things as in Generic Letter 85-09. However, the wording was different and was likely to cause confusion. The licensee's proposed amendments would eliminate the confusion and adopt the substance of Generic Letter 85-09. The amendments are therefore acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. We have previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 19, 1988

Principal Contributor: Peter S. Tam