

MAY 24 1988

Docket No. 50-334

Mr. J. D. Sieber, Vice President  
Duquesne Light Company  
Nuclear Group  
Post Office Box 4  
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Dear Mr. Sieber:

SUBJECT: BEAVER VALLEY UNIT 1 - CORRECTION OF ADMINISTRATIVE ERROR IN  
AMENDMENT NO. 121 (TAC 66674)

Amendment No. 121 was issued on March 14, 1988. However, recently we noticed that page 3/4 11-14 was not included in the amendment package, even though the safety evaluation addressed that page and found the proposed change acceptable.

Enclosed please find page 3/4 11-14. We apologize for any inconvenience this may have caused.

Sincerely,

**"ORIGINAL SIGNED BY"**

Peter S. Tam, Project Manager  
Project Directorate I-4  
Division of Reactor Projects I/II

Enclosure:  
As stated

cc: w/enclosure  
See next page

LA:PDI-4  
SNorris  
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TABLE NOTATION

- a. The Lower Limit of Detection (LLD) is defined in Table Notation (a) of Table 4.11-1 of Specification 4.11.1.1.
- b. When reactor coolant system activity exceeds the limits stated in Specification 3.4.8, analyses shall be performed once every 24 hours during startup, shutdown and 25% load changes and 72 hours after achieving the maximum steady state power operation unless continuous monitoring is provided.
- c. Tritium grab samples shall be taken at least once per 24 hours (from the appropriate ventilation release pathway) when the refueling canal is flooded.
- d. Samples shall be changed at least once per 7 days and analyses shall be completed within 48 hours after changing (or after removal from sampler). Sampling and analyses shall also be performed at least once per 24 hours, during startup, shutdown and 25% load changes and 72 hours after achieving the maximum steady state power operation when RCS activity exceeds the limits in Specification 3.4.8 unless continuous monitoring is provided. When samples collected for 24 hours are analyzed, the corresponding LLD's may be increased by a factor of 10.
- e. Tritium grab samples shall be taken at least once per 7 days from the ventilation exhaust from the spent fuel pool area, whenever spent fuel is in the spent fuel pool.
- f. The average ratio of the sample flow rate to the sampled stream flow rate shall be known for the time period covered by each dose or dose rate calculation made in accordance with Specification 3.11.2.1, 3.11.2.2 and 3.11.2.3.
- g. The principal gamma emitters for which the LLD specification will apply are exclusively the following radionuclides: Kr-87, Kr-88, Xe-133, Xe-133m, Xe-135, and Xe-138 for gaseous emissions and Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137, Ce-141, and Ce-144 for particulate emissions. This list does not mean that only these nuclides are to be detected and reported. Other peaks which are measurable and identifiable, together with the above nuclides, shall also be identified and reported. Nuclides which are below the LLD for the analyses should not be reported as being present at the LLD level for that nuclide. When unusual circumstances result in LLD's higher than required, the reasons shall be documented in the semi-annual effluent report.