

March 13, 1989

Docket No. 50-334

Mr. J. D. Sieber, Vice President
Nuclear Group
Duquesne Light Company
Post Office Box 4
Shippingport, Pennsylvania 15077

Dear Mr. Sieber:

SUBJECT: BEAVER VALLEY UNIT 1 - ISSUANCE OF AMENDMENT (TAC NOS. 69364 AND 45114)

The Commission has issued the enclosed Amendment No. 137 to Facility Operating License No. DPR-66 for Beaver Valley Power Station, Unit No. 1, in response to your application dated September 7, 1988.

The amendment imposes requirements on the incore thermocouples and the reactor vessel level indicating system (RVLIS). In addition, the existing specification on the subcooling margin monitor is revised to require at least one channel operable (from zero required).

We note that the specification issued on the RVLIS is not strictly in compliance with the guidelines of Generic Letter 83-37. However, even though your proposed specification does not fully conform with staff guidance, it does provide sufficient control of the RULIS to assure safe plant operation.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

/s/

Peter S. Tam, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 137 to DPR-66
2. Safety Evaluation

cc w/enclosures:
See next page

[5520 Document Name: AMEND TAC 69364]

LA:PDI-4
SNorris*
02/22/89

PM:PDI-4
PTam:lm *PST*
03/13/89

PS:PDI-4
JSto *sz*
02/13/89

OGC
BBordenick*
02/27/89

BC:SRXB
MHodges*
2/22/89

*See previous concurrence. Third paragraph was changed to accommodate comment by J. Scinto of OGC.

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We note that the specification issued on the RVLIS is not strictly in compliance with the guidelines of Generic Letter 83-37. However, the specification is issued since none was in existence. We will continue to evaluate the need to upgrade the RVLIS specification and may take future unilateral actions to impose a more stringent requirement.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

Peter S. Tam, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

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- 2. Safety Evaluation

cc w/enclosures:
See next page

*all noted
dated 2/8/89*

[5520 Document Name: AMEND TAC 69364]

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PTam:lm
02/22/89

PD:PDI-4
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02/27/89

must
BC:SRXB
MHodges
02/22/89

Mr. J. Sieber
Duquesne Light Company

Beaver Valley Power Station
Units 1 & 2

cc:

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AMENDMENT NO. 137 TO FACILITY OPERATING LICENSE NO. DPR-66

DATED: March 13, 1989

Docket File

NRC PDR

Local PDR

Gray File

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B. Boger (14A2)

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ACRS (10)

GPA/PA

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 137
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated September 7, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 137, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 13, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 137

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of Appendix A (Technical Specifications) with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 3-51

3/4 3-52

Insert

3/4 3-51

3/4 3-52

TABLE 3.3-11

ACCIDENT MONITORING INSTRUMENTATION

	<u>TOTAL NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Pressurizer Water Level	3	2
2. Auxiliary Feedwater Flow Rate	1 per steam generator	1 per steam generator
3. Reactor Coolant System Subcooling Margin Monitor	1	1
4. PORV Acoustical Detector Position Indicator	2/valve*	1/valve
5. PORV Limit Switch Position Indicator	1/valve	0/valve
6. PORV Block Valve Limit Switch Position Indicator	1/valve	0/valve
7. Safety Valve Acoustical Detector Position Indicator	2/valve*	1/valve
8. Safety Valve Temperature Detector Position Indicator	1/valve	0/valve
9. Containment Sump Wide Range Water Level	2	1
10. Containment Wide-Range Pressure	2	0
11. In-Core Thermocouples (Core-Exit Thermocouples)	4/core quadrant	2/core quadrant
12. Reactor Vessel Level Indicating System	1	1

* One Detector Active, Second Detector Passive

TABLE 4.3-7ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Pressurizer Water Level	M	R
2. Auxiliary Feedwater Flow Rate	S/U(1)	R
3. Reactor Coolant System Subcooling Margin	M	R
4. PORV Acoustical Detector Position Indicator	M	R
5. PORV Limit Switch Position Indicator	M	R
6. PORV Block Valve Limit Switch Position Indicator	M	R
7. Safety Valve Acoustical Detector Position Indicator	M	R
8. Safety Valve Temperature Detector Position Indicator	M	R
9. PORV Control Pressure Channels (PT-RC-444, 445)	M	R
10. Containment Sump Wide-Range Water Level	M	R
11. Containment Wide-Range Pressure	N/A	R
12. In-Core Thermocouples (Core-Exit Thermocouples)	M	R
13. Reactor Vessel Level Indicating System	M	R

- (1) Channel check to be performed in conjunction with Surveillance Requirement 4.7.1.2.a.9 following an extended plant outage.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 137 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY
OHIO EDISON COMPANY
PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

INTRODUCTION

We have completed review of the design of the inadequate core cooling instrumentation (ICCI) at Beaver Valley Unit 1, and have transmitted results of our review by letter dated May 12, 1987. In that letter we stated that the proposed design satisfies the requirements of NUREG-0737, item II.F.2. At the end of the sixth refueling outage, the ICCI was completely installed and its operability was established (Duquesne Light Company letter dated March 4, 1988).

In our safety evaluation, we stated that revised technical specifications to cover the ICCI, in accordance with the guidance of Generic Letter (GL) 83-37, would be needed. In response, Duquesne Light Company (the licensee, acting as agent for the above utilities) submitted an amendment request on September 7, 1988.

DISCUSSION AND EVALUATION

a) Table 3.3-11, Accident Monitoring Instrumentation

The licensee proposed to include in this table the incore thermocouples (core-exit thermocouples), with two channels minimum per core quadrant required operable out of the total of four channels per core quadrant. This fully complies with the guidance of GL 83-37 and is acceptable.

The licensee proposed the minimum channels operable for the reactor coolant subcooling margin monitor and the reactor vessel level indicating system (RVLIS) to be one, which is consistent with GL 83-37. Two reactor coolant system subcooling margin monitor channels and two RVLIS channels are installed and available to satisfy the requirements of this specification. However, Table 3.3-11 only specifies one for the total

number of channels. This deviates from the two specified in GL 83-37. This is based on the facts that these instruments perform no automatic actions, any procedural actions keyed to specific values readable on these instruments have alternate means available for determining subcooling margin, and the emergency operating procedures instruct the operators on what actions to take either with or without RVLIS. Therefore, the licensee maintained that continued plant operation need not be limited when one channel of these instruments is inoperable. Though those instruments provide additional information, they are not critical to the operation of the plant. They have been added to the technical specifications to ensure periodic calibration is performed and at least one channel is available. The licensee's proposed requirement of one does not meet the guidance of GL 83-37. However, in light of the fact that there is currently no technical specification on the RVLIS, and the licensee requested the amendment under 10 CFR 50.90, we have determined that it is prudent to issue the requirement as requested. Even though the proposed specification does not fully conform to the staff guidance, it does provide sufficient control of the RVLIS to assure safe plant operation.

The licensee also proposed to upgrade the current minimum channel requirement for the RCS subcooling margin monitor from zero to one. This brings the requirement into conformance with the guidance of GL 83-37 and is acceptable.

(b) Table 4.3-7, Accident Monitoring Instrumentation Surveillance Requirements

The licensee proposed to incorporate surveillance requirements for the RVLIS, core-exit thermocouples and subcooling margin monitor in accordance with the guidance of GL 83-37. These are acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment changes requirements with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. We have previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations and, (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 13, 1989

Principal Contributor: Peter S. Tam