

November 8, 1984

Docket No. 50-334

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Mr. J. J. Carey, Vice President
Nuclear Division
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

Dear Mr. Carey:

The Commission has issued the enclosed Amendment No.82 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your applications dated July 14, 1983 and dated May 7, 1984. While your July 14, 1983 application addresses a number of issues, this amendment addresses only the issues described below.

The amendment changes the Technical Specifications for Beaver Valley Unit No. 1 to (1) allow continued plant operation with one air lock door inoperable under certain conditions, (2) add new surveillance requirements to the emergency air lock, and (3) change the existing air lock surveillance requirements. In addition, the amendment also corrects a typographical error on Page 3/4 6-10 of the Technical Specifications.

The requested change to Specification 4.6.1.3(b) would involve an exemption to Appendix J of 10 CFR 50, and will be addressed by a future action.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

Peter S. Tam, Project Manager
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:

1. Amendment No. 82 to DPR-66
2. Safety Evaluation

cc w/enclosures:
See next page

ORB#1:DL
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10/2/84

ORB#1:DL
PTam
10/19/84

C. ORB#1:DL
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10/27/84

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10/25/84

AD/OR:DL
GLafinas
11/7/84

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J. Scinto's concern has been addressed by addition of a sentence in first paragraph of SER.

*ok 11/7/84
Add with that
amendment manual
except for
letter is not
addressed
BT 11/7/84*

No See Note 11/7/84

Mr. J. J. Carey
Duquesne Light Company

cc: Mr. W. S. Lacey
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Beaver Valley Power Station
Unit 1

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Beaver Valley Power Station
Unit 1

- 2 -

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE.

Amendment No.82
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated July 14, 1983 and May 7, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 82, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This amendment is effective on issuance, to be implemented no later than 30 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance:

November 8, 1984

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 82 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 6-5	3/4 6-5
• --	3/4 6-5a
3/4 6-10	3/4 6-10

CONTAINMENT SYSTEMS

CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

3.6.1.3 Each containment air lock shall be OPERABLE with:

- a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment; then at least one air lock door shall be closed, and
- b. An overall air lock leakage rate of less than or equal to $0.05 L_a$ at P_a (38.3 psig).

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With one containment air lock door inoperable:
 1. Maintain the associated OPERABLE air lock door closed and either restore the associated inoperable air lock door to OPERABLE status within 24 hours or lock the associated OPERABLE air lock door closed.
 2. Operation may then continue until performance of the next required overall air lock leakage test provided that the associated OPERABLE air lock door is verified to be locked closed at least once per 31 days.
 3. Otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
 4. The provisions of Specification 3.0.4 are not applicable.
- b. With a containment air lock inoperable, except as the result of an inoperable air lock door, maintain at least one air lock door closed; restore the inoperable air lock to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:

a. Within 72 hours following each containment entry, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying no detectable seal leakage when the gap between the door seals is pressurized for at least 2 minutes to:

1. Personnel airlock ≥ 38.3 psig
2. Emergency air lock ≥ 10.0 psig

or, by quantifying the total air lock leakage to insure the requirements of 3.6.1.3.b are met.

b. At least once per 6 months by conducting an overall air lock leakage test at P (38.3 psig) and by verifying that the overall air lock leakage rate is within its limit, and

c. At least once per 18 months during shutdown by verifying:

1. Only one door in each air lock can be opened at a time, and
2. No detectable seal leakage when the volume between the emergency air lock shaft seals is pressurized to greater than or equal to 38.3 psig for at least 2 minutes.

The provisions of Specification 4.0.2 are not applicable.

CONTAINMENT SYSTEMS

CONTAINMENT STRUCTURAL INTEGRITY

LIMITING CONDITIONS FOR OPERATION

3.6.1.6 The structural integrity of the containment shall be maintained at a level consistent with the acceptance criteria in Specification 4.6.1.6.1.

APPLICABILITY: MDES 1, 2, 3 and 4.

ACTION:

With the structural integrity of the containment not conforming to the above requirements, restore the structural integrity to within the limits prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.6.1 Liner Plate and Concrete: The structural integrity of the containment liner plate and concrete shall be determined during the shutdown for each Type A containment leakage rate test (reference Specification 4.6.1.2) by:

- a. a visual inspection of the accessible surfaces and verifying no apparent changes in appearance or other abnormal degradation.
- b. a visual inspection of accessible containment liner test channels prior to each Type A containment leakage rate test. Any containment liner test channel which is found to be damaged to the extent that channel integrity is impaired or which is discovered with a vent plug removed, shall be removed and a protective coating shall be applied to the liner in that area.
- c. a visual inspection of the dome area prior to each Type A containment leakage rate test to insure the integrity of the protective coating. If a loss of integrity of the protective coating is observed, any vent plug to a test channel which may be in the area where the protective coating has failed shall be seal welded and then the protective coating shall be repaired.

4.6.1.6.2 Reports An initial report of any abnormal degradation of the containment structure detected during the above required tests and inspections shall be made within 10 days after completion of the surveillance requirements of this specification, and the detailed report shall be submitted pursuant to Specification 6.9.1 within 90 days after completion. This report shall include a description of the condition of the liner plate and concrete, the inspection procedure, the tolerances on cracking and the corrective actions taken.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 82 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

Introduction

By letters dated July 14, 1983 and May 7, 1984, Duquesne Light Company (the licensee) applied for an amendment to Operating License DPR-66 to make a number of changes in the Technical Specifications. The following is an evaluation of the proposed changes to the Technical Specifications pertaining to containment air locks and the recently installed emergency air lock.

Discussion and Evaluation

The licensee proposes to revise the Action (Section 3.6.1.3) and Surveillance Requirements (Section 4.6.1.3) for the containment air locks.

Section 3.6.1.3

The Action statement would be changed to allow continued plant operation with one air lock door inoperable, provided that the operable air lock door is locked closed and verified to be locked closed at least once per 31 days. For the personnel air lock, a door will be locked closed by deactivating the hydraulically operated latches; for the emergency air lock this will be accomplished by physically locking the emergency egress air lock doors. Allowing continued plant operation with one air lock door inoperable for up to six months is acceptable since containment boundary integrity will be preserved by having the redundant, operable air lock door administratively controlled closed. (Locking the personnel air lock door would still leave the emergency manhole available for emergency escape from inside the containment. See Updated FSAR Section 5.2.4.8).

Section 4.6.1.3

This Section would be changed as follows: Subsection (a) would be amended to provide the option of performing door seal leak tests or an overall air lock leak test within 72 hours following each containment entry. A test pressure of 10 psig or greater for the emergency air lock door seals, and a door seal test pressure of greater than or equal to Pa (38.3 psig) for the personnel air lock are specified. The licensee states that the lower test

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pressure for the emergency air lock door seal is the value recommended by the air lock manufacturer. Since the emergency air lock door is not equipped with strongbacks, testing at a pressure approaching Pa will tend to unseat the door. The proposed changes are acceptable.

Subsection (c) would be amended to establish an additional test requirement for the emergency air lock, in that the emergency air lock shaft seals would be tested at least once per 18 months during shutdown to provide added assurance of seal integrity. The acceptance criterion of this test is that there be no detectable leakage when the volume between the emergency airlock shaft seals is pressurized to greater than or equal to Pa (38.2 psig) for at least 2 minutes. The proposed criterion is acceptable.

The requested change to Subsection (b) would involve an exemption to the requirements of Appendix J to 10 CFR Part 50, Section III.D.2.(b)(ii), and will be addressed by a future action.

In summary, the licensee's proposed changes to Sections 3.6.1.3 and 4.6.1.3 of the Technical Specifications are administrative in nature and do not involve physical changes to the air locks. There is adequate assurance that containment boundary integrity will be preserved during plant operation, and that the leakage integrity of the air locks will be maintained. Therefore, we conclude that the proposed changes to the Technical Specifications are acceptable, as stated above.

Environmental Consideration

This amendment involves a change in the administrative procedure and requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities

will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 8, 1984

Principal Contributor:

J. Guo