August 12, 1986

Docket No. 50-334

Mr. J. J. Carey, Vice President
Nuclear Group
Duquesne Light Company
Post Office Box 4
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Dear Mr. Carey:

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Subject: Issuance of Amendment (Licensing Action TAC 54382 and 54516)

The Commission has issued the enclosed Amendment No.105to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application dated October 8, 1984, and revised by letter dated February 3, 1986.

The amendment changes the Technical Specifications for Beaver Valley Unit No. 1 to comply with the requirements (NUREG-0737) imposed by the Commission as a result of the Three Mile Island accident. The changes are patterned after the staff's standard technical specifications transmitted by Generic Letter 83-37 and are related to the containment sump water level and pressure instruments, the reactor coolant vent system, the containment hydrogen analyzer and the post-accident monitoring program.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/

Peter S. Tam, Project Manager
PWR Project Directorate #2
Division of PWR Licensing-A

Enclosures:
1. Amendment No.105to DPR-66
2. Safety Evaluation

cc w/enclosures:
See next page

LA:PAD#2	PM:PAD#2	D:PAD#2	OELD
DMiller	PTam:hc	LRubenstein	
8/11/86	8/17/86	8/ /86	8/25/86

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.105
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated October 8, 1984, and revised by letter dated February 3, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.105 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This amendment is effective on issuance, to be implemented no later than 60 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

L.S. Rubenstein

Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 12, 1986

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 105 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

Remove Pages	Insert Pages
-	3/4 4-32
-	3/4 4-33
-	B 3/4 4-11
3/4 3-51	3/4 3-51
3/4 3-52	3/4 3-52
3/4 6-20	3/4 6-20
6-13	6-13

REACTOR COOLANT SYSTEM VENTS

LIMITING CONDITION FOR OPERATION

3.4.12 All reactor coolant system vent valves, powered from emergency buses, shall be OPERABLE* and closed** for each vent path from the following locations:

a. Reactor Vessel Head
b. Pressurizer Steam Space

APPLICABILITY: MODES 1, 2, 3 and 4

ACTION:

a. With at least one vent path from each of the above locations OPERABLE and one or more power operated vent valves inoperable, STARTUP and/or POWER OPERATION may continue provided the inoperable valve(s) is maintained closed with power removed or with the manual isolation valve closed. Power operation may continue until the next scheduled outage, at which time all reactor coolant system vent valves shall be OPERABLE prior to entry into MODE 1. The provisions of Specification 3.0.4 are not applicable.

b. With all vent paths from one of the above locations inoperable, maintain the inoperable valves closed with power removed or with the manual isolation valves closed, restore at least one of the inoperable vent paths to OPERABLE status within 30 days, or, be in HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

c. With all vent paths from both of the above locations inoperable, maintain the inoperable valves closed with power removed or close the manual isolation valves, and restore at least one vent path from one of the above locations to OPERABLE status within 72 hours or be in HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.12 Each reactor coolant system vent path shall be demonstrated OPERABLE at least once per 18 months by:

1. Verifying all manual isolation valves in each vent path are locked or sealed in the open position.

2. Cycling each valve in the vent path through at least one complete cycle of full travel from the control room.

3. Verifying flow through the reactor coolant system vent path to the Pressurizer Relief Tank.

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* For purposes of this specification an inoperable vent valve is defined as: a valve which exhibits leakage in excess of Specification 3.4.6.2 limits, or cannot be opened and closed on demand, or does not have its normal emergency power supply OPERABLE.

** These valves may be operated for required venting operations and leak testing in Modes 3 and 4.

3/4.4.12 REACTOR COOLANT SYSTEM VENTS

Reactor Coolant System Vents are provided to exhaust noncondensible gases and/or steam from the primary system that could inhibit natural circulation core cooling. The OPERABILITY of at least one reactor coolant system vent path from the reactor vessel head and the pressurizer steam space, ensures the capability exists to perform this function.

The valve redundancy of the reactor coolant system vent paths serves to minimize the probability of inadvertent or irreversible actuation while ensuring that a single failure of a vent valve, power supply or control system does not prevent isolation of the vent path.

The function, capabilities, and testing requirements of the reactor coolant system vent systems are consistent with the requirements of Item II.B.1 of NUREG-0737, "Clarification of TMI Action Plan Requirements", November 1980.

TABLE 3.3-11

ACCIDENT MONITORING INSTRUMENTATION

	TOTAL NO. OF CHANNELS	MINIMUM CHANNELS OPERABLE
1. Pressurizer Water Level	3	2
2. Auxiliary Feedwater Flow Rate	1 per steam generator	1 per steam generator
3. Reactor Coolant System Subcooling Margin Monitor	1	0
4. PORV Accoustical Detector Position Indicator	2/valve*	1/valve
5. PORV Limit Switch Position Indicator	1/valve	0/valve
6. PORV Block Valve Limit Switch Position Indicator	1/valve	0/valve
7. Safety Valve Accoustical Detector Position Indicator	2/valve*	1/valve
8. Safety Valve Temperature Detector Position Indicator	1/valve	0/valve
9. Containment Sump Wide Range Water Level	2	1
10. Containment Wide-Range Pressure	2	0

* One Detector Active, Second Detector Passive

TABLE 4.3-7

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

	INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION
1.	Pressurizer Water Level	M	R
2.	Auxiliary Feedwater Flow Rate	S/U(1)	R
3.	Reactor Coolant System Subcooling Margin	M	R
4.	PORV Accoustical Detector Position Indicator	M	R
5.	PORV Limit Switch Position Indicator	M	R
6.	PORV Block Valve Limit Switch Position Indicator	M	R
7.	Safety Valve Accoustical Detector Position Indicator	M	R
8.	Safety Valve Temperature Detector Position Indicator	M	R
9.	PORV Control Pressure Channels (PT-RC-444, 445)	M	R
10.	Containment Sump Wide-Range Water Level	M	R
11.	Containment Wide-Range Pressure	N/A	R

(1) Channel check to be performed in conjunction with Surveillance Requirement 4.7.1.2.a.9 following an extended plant outage.

3/4.6.4 COMBUSTIBLE GAS CONTROL

HYDROGEN ANALYZERS

LIMITING CONDITION FOR OPERATION

3.6.4.1 Two separate and independent wide-range containment hydrogen analyzers shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

ACTION:

a. With one wide-range hydrogen analyzer inoperable, restore the inoperable analyzer to OPERABLE status within 30 days or be in HOT STANDBY within the next 12 hours.

b. With both wide-range hydrogen analyzers inoperable, restore at least one wide-range hydrogen analyzer to OPERABLE status within 72 hours or be in HOT STANDBY within the next 12 hours.

SURVEILLANCE REQUIREMENTS

4.6.4.1 Each hydrogen analyzer shall be demonstrated OPERABLE at least once per 92 days on a STAGGERED TEST BASIS by:

a. Performing a CHANNEL CALIBRATION using sample gases containing:

1. One volume percent hydrogen, balance nitrogen, and

2. Four volume percent hydrogen, balance nitrogen.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

a. The intent of the original procedure is not altered.

b. The change is approved by two (2) members of the plant management staff, at least one (1) of whom holds a Senior Reactor Operator's License on the unit affected.

c. The change is documented, reviewed by the OSC and approved by the Plant Superintendent within 14 days of implementation.

6.8.4 A Post-Accident monitoring program shall be established, implemented, and maintained:

A program which will provide the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples following an accident. The program shall include the following:

(i) Training of personnel,
(ii) Procedures for sampling and analysis, and
(iii) Provisions for maintenance of sampling and analysis equipment.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

STARTUP REPORTS

6.9.1.1 A summary report of plant startup and power escalation testing will be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

6.9.1.2 The startup report shall address each of the tests identified in the FSAR and shall include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details requested in license conditions based on other commitments shall be included in this report.

6.9.1.3 Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 105 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

INTRODUCTION

By letter dated October 8, 1984, and revised by letter dated February 3, 1986, Duquesne Light Company (the licensee) submitted a proposed amendment to the technical specifications (Appendix A of Operating License No. DPR-66) for Beaver Valley Power Station, Unit No. 1, to add requirements and bases for the reactor coolant system vents, to add instruments in the accident monitoring instrumentation section, to revise requirements for combustible gas control, and to add administrative requirements for a post-accident monitoring program. The four proposed changes address technical specification changes requested by the NRC in Generic Letter 83-37. No existing requirements are to be deleted. We have reviewed the requested changes, and the results are as follows.

EVALUATION AND DISCUSSION

10 CFR 50.44(c)(3)(iii) specifies requirements for reactor coolant system (RCS) vents. By letter dated September 8, 1983, the NRC issued a safety evaluation (SE) which found the licensee-proposed RCS vents to be technically acceptable. The licensee proposes to incorporate limiting conditions for operation (LCOs), surveillance requirements, and bases for the RCS vents which are consistent with Generic Letter 83-37 and the September 8, 1983 SE. The proposed RCS vent requirements and bases are acceptable.

NUREG-0737 requires the installation of wide-range instruments to monitor containment pressure and water level under accident conditions. By letter dated May 17, 1983, the NRC issued an SE which found the licensee-proposed instruments for these parameters to be technically acceptable. The licensee proposed to add these instruments to the existing tables for Accident Monitoring Instrumentation (Table 3.3-11) and Accident Monitoring Instrumentation Surveillance Requirements (Table 4.3-7). This is consistent with the May 17, 1983 SE and Generic Letter 83-37. The proposed addition of these requirements is acceptable.

Also, the licensee proposed to add an additional instrument to Table 3.3-11, in order to be consistent with Table 4.3-7. Specifically, Amendment No. 45 added a surveillance requirement to calibrate the power-operated relief valve (PORV) control channels to paragraph 4.4.11.1.b and added the control channels to Table 4.3-7, Accident Monitoring Instrumentation Surveillance Requirements, for listing of frequency. Since the control channels are currently listed in Table 4.3-7 but not in Table 3.3-11, which contains the corresponding LCOs, the licensee proposed adding the control channels to Table 3.3-11 for consistency. However, in an April 24, 1986 telephone conversation, the licensee representative, S. Sovick, agreed to delete this proposed addition, because it would apply two conflicting LCOs to the PORV control channels.

NUREG-0737 requires the installation of wide-range instruments to monitor containment hydrogen under accident conditions. By letter dated May 17, 1983, the NRC issued an SE which found the licensee-proposed instruments for hydrogen monitoring to be technically acceptable. Currently, Section 3/4.6.4, Combustible Gas Control, provides requirements for containment hydrogen analyzers. The licensee proposes to revise the existing requirements so that it is clear that the requirements apply to the wide-range hydrogen analyzers installed as part of NUREG-0737 and to add an additional action statement. This is consistent with Generic Letter 83-37 and the May 17, 1983 SE. The proposed revision of the hydrogen monitoring requirements is acceptable.

NUREG-0737 requires the implementation of a program for post-accident sampling, including training, procedures, and equipment maintenance. Generic Letter 83-37 provided an acceptable technical specification change for the administrative requirements for this program. The licensee proposes to add a new Section 6.8.4, consistent with Generic Letter 83-37, to accomplish this. The proposed addition of administrative requirements for the post-accident sampling program is acceptable.

We have evaluated the proposed changes to the technical specifications and conclude that these changes do not involve any physical change to the plant's safety-related structures, systems or components. Further, these changes do not increase the likelihood of a malfunction of safety-related equipment, or increase the consequences of an accident previously analyzed or create the possibility of a malfunction different from those previously evaluated. Therefore, we find the licensee's requested changes to be acceptable.

ENVIRONMENTAL CONSIDERATIONS

This amendment involves changes in the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20 or change in a surveillance requirement. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR z51.22(c)(9).

This amendment also involves changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, with respect to these items, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(10). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: August 12, 1986

Principal Contributor:

Glenn W. Meyer