

September 30, 1983

Docket No. 50-334

Mr. J. J. Carey, Vice President
Duquesne Light Company
Nuclear Division
Post Office Box 4
Shippingport, Pennsylvania 15077

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Dear Mr. Carey:

The Commission has issued the enclosed Amendment No.72 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated April 12, 1983.

The amendment clarifies the intent of Amendment No. 49, dated March 30, 1982, by adding two footnotes to Sections 3.7.12 and 4.7.12. The footnotes do not effect the technical requirement for the surveillance of snubbers. Footnote 1 defines the systems that require operable snubbers during modes 5 & 6, while footnote 2 specifies the way to determine the baseline for snubber service life.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

Peter S. Tam, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No.72 to DPR-66
2. Safety Evaluation

cc: w/enclosures
See next page

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Mr. S. J. Carey
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 72
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated April 12, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 72, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 30, 1983

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 72 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

Remove Pages

3/4 7-26

3/4 7-29

Insert Pages

3/4 7-26

3/4 7-29

PLANT SYSTEMS

3/4.7.12 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.12 All snubbers listed in Tables 3.7-4a and 3.7-4b shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4. (MODES 5 and 6 for snubbers located on systems** required OPERABLE in those MODES).

ACTION:

With one or more snubbers inoperable, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.12.c on the supported component or declare the supported system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.12 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

a. Visual Inspections

The first inservice visual inspection of snubbers shall be performed after four months but within 10 months of commencing POWER OPERATION and shall include all snubbers listed in Tables 3.7-4a and 3.7-4b. If less than two (2) snubbers are found inoperable during the first inservice visual inspection, the second inservice visual inspection shall be performed 12 months \pm 25% from the date of the first inspection. Otherwise, subsequent visual inspections shall be performed in accordance with the following schedule:

<u>No. Inoperable Snubbers per Inspection Period</u>	<u>Subsequent Visual Inspection Period* #</u>
0	18 months \pm 25%
1	12 months \pm 25%
2	6 months \pm 25%
3,4	124 days \pm 25%
5,6,7	62 days \pm 25%
8 or more	31 days \pm 25%

The snubbers may be categorized into two groups: those accessible and those inaccessible during reactor operation. Each group may be inspected independently in accordance with the above schedule.

* The inspection interval shall not be lengthened more than one step at a time.

The provisions of Specification 4.0.2 are not applicable.

**These systems are defined as those portions or subsystems required to prevent releases in excess of 10 CFR 100 limits.

SURVEILLANCE REQUIREMENTS (Continued)

e. Mechanical Snubbers Functional Test Acceptance Criteria

The mechanical snubber functional test shall verify that:

1. The force that initiates free movement of the snubber rod in either tension or compression is less than the specified maximum drag force.
2. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.
3. Snubber release rate, where required, is within the specified range in compression or tension. For snubbers specifically required not to displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

f. Snubber Service Life Monitoring*

A record of the service life of each snubber, the date at which the designated service life commences and the installation and maintenance records on which the designated service life is based shall be maintained as required by Specification 6.10.2.m.

Concurrent with the first inservice visual inspection and at least once per 18 months thereafter, the installation and maintenance records for each snubber listed in Tables 3.7-4a and 3.7-4b shall be reviewed to verify that the indicated service life has not been exceeded or will not be exceeded prior to the next scheduled snubber service life review. If the indicated service life will be exceeded prior to the next scheduled snubber service life review, the snubber service life shall be reevaluated or the snubber shall be replaced or reconditioned so as to extend its service life beyond the date of the next scheduled service life review. This reevaluation, replacement or reconditioning shall be indicated in the records.

* For purposes of establishing a baseline for the determination of service life monitoring, this program will be implemented over 3 successive refueling periods.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 72 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

Introduction

On April 12, 1983, the Duquesne Light Company submitted a document for the Beaver Valley Power Station, Unit No. 1. This document proposes two footnote additions to clarify the intent of Amendment 49 to the Technical Specifications applicable to snubbers for the above mentioned plant.

Evaluation and Discussion

Two footnote additions are proposed for Amendment 49 to the Technical Specifications for the Beaver Valley Power Station, Unit No. 1. They are:

1. A footnote to read: "These systems are defined as those portions of subsystems required to prevent releases in excess of 10 CFR 100 limits" to be added to the applicability statement for Modes 5 and 6 in Section 3.7.12, "Snubber Operability."
2. A footnote to read: "For purposes of establishing a baseline for the determination of service life monitoring, this program will be implemented over three successive refueling periods" is to be added to the surveillance requirement 4.7.12.f "Snubber Service Life Monitoring."

The reason to maintain snubber operability during Modes 5 (cold shutdown) and 6 (refueling) is that, should a dynamic or seismic event occur during that period of time, operable snubbers will offer protection to those systems which could release radioactive materials beyond the 10 CFR 100 limits. The footnote proposed in paragraph 1 clarifies this intent in full and, therefore, is acceptable.

The purpose of service life monitoring is to provide a statistical basis to implement the replacement and repair program at reasonable intervals. The required baseline of service life could be provided by the manufacturer or could be determined by other means. Duquesne Light Company elected to determine this baseline by successively monitoring the inspection results over three refueling outages, or in approximately five years time. The proposal is, therefore, reasonable and acceptable.

Summary

The two proposed footnotes do not affect the technical requirement for the surveillance of snubbers in Beaver Valley Power Station, Unit No. 1. Footnote 1 defines the systems that require operable snubbers during Modes 5 and 6 in an acceptable way. Footnote 2 specifies the way to determine the baseline for snubber service life. They clarify the Technical Specifications and are acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
(2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to common defense and security or to the health and safety of the public.

Dated: September 30, 1983

Principal Contributor:
H. Shaw