

Docket No. 50-334 December 31, 1984

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Mr. J. J. Carey, Vice President  
Nuclear Division  
Duquesne Light Company  
Post Office Box 4  
Shippingport, PA 15077

Dear Mr. Carey:

SUBJECT: ISSUANCE OF AMENDMENT (LICENSING ACTION TAC 55312)

The Commission has issued the enclosed Amendment No.87 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application dated June 25, 1984.

The amendment changes the Technical Specifications for Beaver Valley Unit No. 1 to detach the schedule for the containment Type A leak test from the schedule of inservice inspection. An exemption from 10 CFR 50, Appendix J, Section III.D.1(a), was granted on December 5, 1984, allowing that this amendment be issued. The new technical specification still requires that Type A leak tests be done at 40± 10-month intervals even though they no longer need to be performed in conjunction with inservice inspection.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

/s/PTam

Peter S. Tam, Project Manager  
Operating Reactors Branch No. 1  
Division of Licensing

Enclosures:

1. Amendments No. 87 to DPR-66
2. Safety Evaluation

cc w/enclosures:

See next page

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12/27/84

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Mr. J. J. Carey  
Duquesne Light Company

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Beaver Valley Power Station  
Unit 1

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.87  
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated June 25, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

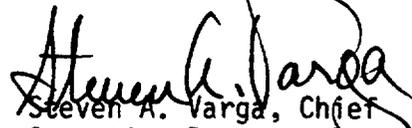
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 87, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This amendment is effective on issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 31, 1984

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 87 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

Remove Pages

3/4 6-2

B 3/4 6-1

Insert Pages

3/4 6-2

B 3/4 6-1

## CONTAINMENT SYSTEMS

### CONTAINMENT LEAKAGE

#### LIMITING CONDITION FOR OPERATION

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3.6.1.2 Containment leakage rates shall be limited to:

- a. An overall integrated leakage rate of:
  1.  $< L_a$ , 0.10 percent by weight of the containment air per 24 hours at  $P_a$ , (38.3 psig), or
- b. A combined leakage rate of  $< 0.60 L_a$  for all penetrations and valves subject to Type B and C tests as identified in Table 3.6-1, when pressurized to  $P_a$ .

APPLICABILITY: MODES 1, 2, 3 and 4.

#### ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding  $0.75 L_a$ , or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding  $0.60 L_a$ , restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

#### SURVEILLANCE REQUIREMENTS

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4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50\* using the methods and provisions of ANSI N45.4-1972:

- a. A Type-A test (Overall Integrated Containment Leakage Rate) shall be conducted at  $40 \pm 10$ -month intervals during shutdown at  $P_a$  (38.3 psig).

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\* Exemption to Appendix J of 10 CFR 50, Section III.D.1(a), granted on December 5, 1984.

## 3/4.6 CONTAINMENT SYSTEMS

### BASES

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#### 3/4.6.1 PRIMARY CONTAINMENT

##### 3/4.6.1.1 CONTAINMENT INTEGRITY

Primary CONTAINMENT INTEGRITY ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the accident analyses. This restriction, in conjunction with the leakage rate limitation, will limit the site boundary radiation doses to within the limits of 10 CFR 100 during accident conditions.

##### 3/4.6.1.2 CONTAINMENT LEAKAGE

The limitations on containment leakage rates ensure that the total containment leakage volume will not exceed the value assumed in the accident analyses at the peak accident pressure,  $P_a$ . As an added conservatism, the measured overall integrated leakage rate  $Q$  is further limited to  $< 0.75 L$  during performance of the periodic test to account for possible degradation of the containment leakage barriers between leakage tests.

The surveillance testing for measuring leakage rates are consistent with the requirements of Appendix "J" of 10 CFR 50.

The exemption to 10 CFR 50 Appendix J.III.D.1(a) allows Type A tests to be conducted on a  $40 \pm 10$ -month schedule, not in conjunction with any ISI tests.

##### 3/4.6.1.3 CONTAINMENT AIR LOCKS

The limitations on closure and leak rate for the containment air locks are required to meet the restrictions on CONTAINMENT INTEGRITY and containment leak rate. Surveillance testing of the air lock seals provides assurance that the overall air lock leakage will not become excessive due to seal damage during the intervals between air lock leakage tests.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 87 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

Introduction

By letter dated June 25, 1984, Duquesne Light Company applied for an amendment to Operating License DPR-66 to change specification 4.6.1.2., "Containment Leakage Surveillance Requirements." The staff's review and evaluation of the application is as follows.

Discussion and Evaluation

Section III.D.1(a) of Appendix J, 10 CFR 50, states that ". . . a set of three Type A tests shall be performed, at approximately equal intervals during each 10-year service period. The third test of each set shall be conducted when the plant is shutdown for the 10-year plant inservice inspections." This requires that containment integrated leak rate tests (CILRTs) be performed at approximately 40-month intervals (in practice,  $40 \pm 10$  month intervals, with the third one in each set performed coincident with performance of the 120-month inservice inspection).

Due to an extended shutdown of eleven months and twenty-one days, the inservice inspection interval has been extended, per the requirements of Section XI of the ASME Boiler and Pressure Vessel Code (See letter, S. A. Varga of NRC to J. J. Carey of Duquesne Light Company, May 21, 1984). It is, therefore, no longer possible to perform the third CILRT within  $40 \pm 10$  months after the second one, and during inservice inspection shutdown.

Duquesne Light Company has requested that Technical Specification 4.6.1.2 be changed, and an exemption from Section III.D.1(a) of Appendix J be granted such that the third of each set of three CILRTs need not be performed in conjunction with the inservice inspection shutdown. The staff has reviewed the request and concluded that since the CILRT need not be performed during the inservice inspection shutdown, the licensee's request is acceptable. An exemption was issued on December 5, 1984. The licensee will continue to perform the CILRTs at interval of  $40 \pm 10$  months but will not need to perform them in conjunction with inservice inspections.

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Environmental Consideration

This amendment involves a change in administrative procedures and requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 31, 1984

Principal Contributor:

J. Guo