

December 31, 1984

Docket No. 50-334

Mr. J. J. Carey, Vice President
Nuclear Division
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

Dear Mr. Carey:

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SUBJECT: ISSUANCE OF AMENDMENT (LICENSING ACTION TAC 55829)

The Commission has issued the enclosed Amendment No.88 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application dated September 5, 1984.

The amendment changes the Technical Specifications for Beaver Valley Unit No. 1 to conform with guidance provided in the Standard Technical Specifications as follows: (1) The requirement to inspect in-containment areas where fire detection instruments have become inoperable has been changed from once per hour to once per 8 hours, or to monitor containment air temperature at selected locations once per hour. (2) The requirement to perform channel functional test on fire detection instruments not accessible during plant operation has been changed from once every 6 months to "during cold shutdown exceeding 24 hours." (3) The reporting requirement for inoperable fire detection instrument has been changed to comply with 10 CFR 50.72 and 50.73.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

/s/PSTam

Peter S. Tam, Project Manager
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:

- 1. Amendments No.88 to DPR-66
- 2. Safety Evaluation

cc w/enclosures:
See next page

ORB#1:DL
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GLainas
12/27/84



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

December 31, 1984

Docket No. 50-334

Mr. J. J. Carey, Vice President
Nuclear Group
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

Dear Mr. Carey:

SUBJECT: ISSUANCE OF AMENDMENT (LICENSING ACTION TAC 55829)

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Sincerely,

A handwritten signature in cursive script that reads "Peter S. Tam".

Peter S. Tam, Project Manager
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:

1. Amendments No. 88 to DPR-66
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. J. J. Carey
Duquesne Light Company

Beaver Valley Power Station
Unit 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 88
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated September 5, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

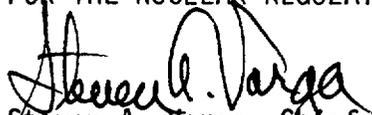
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 88, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This amendment is effective on issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 31, 1984

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 88 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

Remove Pages

3/4 3-47
B 3/4 3-3

Insert Pages

3/4 3-47
B 3/4 3-3

FIRE DETECTION INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.6 As a minimum, the fire detection instrumentation for each fire detection zone shown in Table 3.3-10 shall be OPERABLE:

APPLICABILITY: Whenever equipment in that fire detection zone is required to be OPERABLE.

ACTION

With the number of OPERABLE fire detection instruments less than required by Table 3.3-10:

- a. Within 1 hour establish a fire watch patrol to inspect the zone(s) with the inoperable instrument(s) at least once per hour, unless the instrument(s) is located inside the containment, then inspect the containment at least once per 8 hours or monitor the containment air temperature at least once per hour at the locations listed in Specification 4.6.1.5.
- b. Restore the inoperable instrument(s) to OPERABLE status within 14 days or, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the instrument(s) to OPERABLE status.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.6.1 Each of the above required fire detection instruments which are accessible during plant operations shall be demonstrated OPERABLE at least once per 6 months by performance of a CHANNEL FUNCTIONAL TEST. Fire detectors which are not accessible during plant operation shall be demonstrated OPERABLE by performance of a CHANNEL FUNCTIONAL TEST during each COLD SHUTDOWN exceeding 24 hours unless performed in the previous 6 months.

4.3.3.6.2 The NFPA Code 72D Class A supervised circuits supervision associated with the detector alarms of each of the above required fire detection instruments shall be demonstrated OPERABLE at least once per 6 months.

4.3.3.6.3 The non-supervised circuits between the local panels in Specification 4.3.3.6.2 and the control room shall be demonstrated OPERABLE at least once per 31 days.

INSTRUMENTATION

BASES

3/4.3.3.5 REMOTE SHUTDOWN INSTRUMENTATION

The OPERABILITY of the remote shutdown instrumentation ensures that sufficient capability is available to permit shutdown and maintenance of HOT STANDBY of the facility from locations outside of the control room. This capability is required in the event control room habitability is lost and is consistent with General Design Criteria 19 of 10 CFR 50.

3/4.3.3.6 FIRE DETECTION INSTRUMENTATION

OPERABILITY of the fire detection instrumentation ensures that adequate warning capability is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to safety-related equipment and is an integral element in the overall facility fire protection program.

In the event that a portion of the fire detection instrumentation is inoperable, the establishment of frequent fire patrols or in-containment air temperature monitoring in the affected areas is required to provide detection capability until the inoperable instrumentation is restored to OPERABILITY.

3/4.3.3.7 CHLORINE DETECTION SYSTEMS

The OPERABILITY of the chlorine detection system ensures that sufficient capability is available to promptly detect and initiate protective action in the event of an accidental chlorine release. This capability is required to protect control room personnel and is consistent with the recommendations of Regulatory Guide 1.95, "Protection of Nuclear Power Plant Control Room Operators Against an Accidental Chlorine Release," February 1975.

3/4.3.3.8 ACCIDENT MONITORING INSTRUMENTATION

The OPERABILITY of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with the recommendations of Regulatory Guide 1.97, "Instrumentation for Light-Water-Cooled Nuclear Plants to Assess Plant Conditions During and Following an Accident," December 1975 and NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations."



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 88 TO FACILITY OPERATING LICENSING NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

INTRODUCTION

By letter dated September 5, 1984, Duquesne Light Company (the licensee) proposed to change the Technical Specifications (Appendix A to the license) by revising Sections 3.3.3.6 and 4.3.3.6.1 to reflect Standard Technical Specifications (NUREG-0452) guidance. The licensee also proposed to revise Section 3.3.3.6 to reflect the revised reporting requirements specified by Generic Letter 83-43. No physical changes to the facility or equipment will be made as a result of these changes.

EVALUATION AND DISCUSSION

Currently, specification 3.3.3.6.a requires the licensee to inspect the zone(s) with inoperable fire detection instruments at least once per hour. The licensee proposed to modify the requirement for instruments that are located inside the containment, such that when they are inoperable, they be inspected once per 8 hours or containment air temperature at selected locations be monitored at least once per hour. The proposed change would bring the subject specification into conformance with guidance in the Standard Technical Specifications.

Specification 4.3.3.6.1 requires that fire detection instruments be demonstrated operable at least once per 6 months by performance of a channel functional test. The licensee proposes to change the requirement such that instruments that are not accessible during normal operation be demonstrated operable by performance of a channel functional test "during each cold shutdown exceeding 24 hours unless performed in the previous 6 months". The requested change would also be in conformance with the Standard Technical Specifications.

Specification 3.3.3.6.b will be modified to require submittal of a Special Report to the NRC if a fire detection instrument has been inoperable for more than 14 days. The report will outline the action taken, the cause of the inoperability and the plans and schedule for restoring the instrument to operable status. This requested change is in conformance with 10 CFR 50.72 and 10 CFR 50.73.

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We have evaluated the proposed changes to the Technical Specifications and conclude that these changes are administrative and do not involve any physical change to the plant's safety-related structures, systems or components. Further, these changes do not increase the likelihood of a malfunction of safety-related equipment, or increase the consequences of an accident previously analyzed or create the possibility of a malfunction different from those previously evaluated. Therefore, based on the above, the staff finds the licensee's requested change to revise the Fire Detection Instruments Technical Specification and Bases to be acceptable.

Environmental Consideration

This amendment involves only changes in administrative procedures and requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 31, 1984

Principal Contributors:

Aristides G. Krasopoulos .