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Docket No.: 50-334

Duquesne Light Company
ATTN: Mr. C. N. Dunn, Vice President
Operations Division
435 Sixth Avenue
Pittsburgh, Pennsylvania 15219

Gentlemen:

The Commission has issued the enclosed Amendment No. 8 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station Unit No. 1. The amendment includes changes to the Technical Specifications in response to your application dated October 28, 1976.

This amendment deletes a satisfied condition from the license and adds provisions to the Technical Specifications relating to the operability of the Auxiliary River Water System to assure a continued water supply in the event of loss of the normal river water system.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:
1. Amendment No. 8
2. Safety Evaluation
3. Federal Register Notice

cc w/enclosures: See next page

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Duquesne Light Company

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Duquesne Light Company

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 8
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duquesne Light Company, filed on behalf of itself, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated October 28, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-66 is hereby amended as indicated below and by changes to the Technical Specifications as indicated in the attachment to this license amendment:

   A. Revise paragraph 2.C.(2) to read as follows:

      (2) Technical Specifications

      The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 8 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

   B. Delete paragraph 2.C.(3) and change the numbers of remaining paragraphs 2.C.(4) through 2.C.(9) to 2.C.(3) through 2.C.(8).

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 14, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 8

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace or revise as indicated the following pages of the Appendix "A" Technical Specifications with the attached pages. Revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Corresponding overleaf pages are also provided to maintain document completeness.

Pages

VII
VIII
XIII
XIV
3/4 7-33
3/4 7-34 (added)
B 3/4 7-7 (added)

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LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

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LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

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BASES

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BASES

TABLE 4.7-4

HYDRAULIC SNUBBER INSPECTION SCHEDULE

NUMBER OF SNUBBERS FOUND INOPERABLE DURING INSPECTION OR DURING INSPECTION INTERVAL*	NEXT REQUIRED INSPECTION INTERVAL**
0	18 months ± 25%
1	12 months ± 25%
2	6 months ± 25%
3 or 4	124 days ± 25%
5, 6, or 7	62 days ± 25%
≥8	31 days ± 25%

* Snubbers may be categorized into two groups, "accessible" and "inaccessible". This categorization shall be based upon the snubber's accessibility for inspection during reactor operation. These two groups may be inspected independently according to the above schedule.

** The required inspection interval shall not be lengthened more than one step at a time.

3/4.7.13 AUXILIARY RIVER WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.13.1 At least one auxiliary river water subsystem shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With less than one ARWS subsystem OPERABLE, restore at least one subsystem to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following thirty hours.

SURVEILLANCE REQUIREMENTS

4.7.13.1 At least one ARWS subsystem shall be demonstrated OPERABLE:

a. At least once per 31 days by:

   1. Starting each pump from its control station.

   2. Verifying that each pump develops at least 60 psig discharge pressure while pumping through its test flow line.

   3. Verifying that each pump operates for at least 15 minutes.

   4. Cycling its power operated discharge valve through at least one complete cycle of full travel.

b. At least once per 18 months during shutdown by starting an Auxiliary River Water System Pump, shutting down one Reactor Plant River Water System Pump, and verifying that the Auxiliary River Water Subsystem provides at least 8000 gpm cooling water to that portion of the Reactor Plant River Water System under test for at least 2 hours.

3/4.7.13 AUXILIARY RIVER WATER SYSTEM

The operability of the ARWS ensures that sufficient cooling capacity is available to bring the reactor to a cold shutdown condition in the event that a barge explosion at the station's intake structure or any other extremely remote event would render all of the normal RIVER WATER SYSTEM supply pumps inoperable.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 8 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION UNIT NO. 1

DOCKET NO. 50-334

Introduction

By letter dated October 28, 1976, Duquesne Light Company (DLC) requested approval of proposed Technical Specifications for Beaver Valley Power Station Unit No. 1 (BVPS-1) relating to the Auxiliary River Water System.

Background

In our Safety Evaluation (October 11, 1974), we concluded that the probability of a gasoline barge impact at the river water intake structure, with resulting explosion of the barge, would be high enough to consider this postulated accident in the design of the intake structure. Since this kind of accident could result in a loss of function of the safety-related pumps and piping located in the intake structure, DLC proposed installing an Auxiliary River Water System to provide the cooling requirements for safe plant shutdown in the event that such an accident causes a loss of river water flow from the pumps located in the intake structure.

In Supplement No. 1 (May 2, 1975) to our Safety Evaluation, we concluded that the design of the Auxiliary River Water System was acceptable provided that the design was modified to provide for automatic actuation of the system on loss of normal river water system flow.

Having reviewed additional information provided by DLC, we concluded, in Supplement No. 2 (October 3, 1975) to our Safety Evaluation, that manual actuation of the Auxiliary River Water System is acceptable since an alternate method of plant cooldown, namely the steam turbine-driven Auxiliary Feedwater System, will be available in the event that the operator fails to accomplish the manual actuation of the Auxiliary River Water System.

DLC proposed to install the Auxiliary River Water System within one year after attaining full commercial power operation. We stated that this schedule was reasonable and therefore acceptable. We were considering that full commercial operation would be attained during the latter part of 1975. However, we determined that delays in achieving full commercial operation should not result in delays in completing the installation of the Auxiliary River Water System. Therefore, we required that the Auxiliary River Water System be completed by December 31, 1976.

In Supplement No. 3 (March 19, 1976) to our Safety Evaluation, we concluded that the probability of occurrence of the barge accident was sufficiently low and that plant operation for a limited amount of time was acceptable.

License condition 2.C.(3) of Facility Operating License No. DPR-66 for Beaver Valley Power Station Unit No. 1 currently states that:

> The licensees shall complete the installation of the auxiliary river water system for operation prior to December 31, 1976, or the facility shall not be operated beyond this date. In the interim period to December 31, 1976, Duquesne Light Company shall maintain an alternate cooling system in a state of readiness and shall test the alternate cooling system each calendar quarter for operability. The alternate cooling system is comprised of two diesel-driven pumps, one gasoline engine-driven pump, and 1000 feet of fire hose. The starter batteries for the pump engines shall be tested monthly. The procedures for installing and testing the alternate cooling system shall be incorporated into the plant emergency procedures.

Technical Specifications relating to limiting conditions for operation and surveillance requirements were proposed by DLC letter dated October 28, 1976.

The Auxiliary River Water System was tested for operability on December 7, 1976.

Evaluation

The evaluation of the design adequacy of the Auxiliary River Water System was completed in our Safety Evaluation (October 11, 1974) and Supplements Nos. 1, 2, and 3 (May 2, 1975, October 3, 1975, and March 19, 1976).

During the testing of the Auxiliary River Water System, a representative of the NRC, Office of Inspection and Enforcement, Region I, was present to verify the completion of installation and witness the operability of the system. The ensuing report (IE Inspection Report No. 50-334/76-26) presents the details of the testing and concludes the system to be acceptable.

We have reviewed the Technical Specifications proposed by DLC and find that the specifications are sufficient to assure a continued water supply in the event of a loss of the normal river water system. We conclude that the Technical Specifications relating to surveillance requirements on the Auxiliary River Water System will provide reasonable assurance of system reliability and are therefore acceptable.

We also conclude that license condition 2.C.(3) of DLC Facility Operating License No. DPR-66 has been satisfied and therefore can be deleted.

Environmental Conclusions

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: February 14, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-334

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 8 to Facility Operating License No. DPR-66, issued to Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees), for operation of the Beaver Valley Power Station Unit No. 1 (the facility) located in Beaver County, Pennsylvania. The amendment is effective as of its date of issuance.

The amendment deletes a satisfied condition from the license and adds provisions to the Technical Specifications relating to the operability of the Auxiliary River Water System to assure a continued water supply in the event of loss of the normal river water system.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated October 28, 1976, (2) Amendment No. 8 to License No. DPR-66, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Beaver Area Memorial Library, 100 College Avenue, Beaver, Pennsylvania.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 14th day of February 1977.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors