

December 23, 1985

Docket No. 50-334

Mr. J. J. Carey, Vice President
Nuclear Division
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

Dear Mr. Carey:

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SUBJECT: ISSUANCE OF AMENDMENT (LICENSING ACTION TAC 59014)

The Commission has issued the enclosed Amendment No.99 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application dated June 17, 1985.

The amendment changes the Technical Specifications for Beaver Valley Unit No. 1 to remove specifications for the iodine sampler cartridges from Tables 3.3-13 and 4.3-13. The monitoring function of these cartridges will continue to be performed by the effluent pathway monitors in accordance with Section 4.11.2 of the Technical Specifications. Therefore, the amendment would not result in removing the requirement for iodine sampling, analysis and reporting from the Technical Specifications.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,



Peter S. Tam, Project Manager
PWR Project Directorate #2

Enclosures:

1. Amendment No.99 to DPR-66
2. Safety Evaluation

cc: w/enclosures
See next page

ORB#1:DL
CParrish *cp*
12/18/85

PAD-2
PTam *PT*
12/13/85


D:PAD-2
LRubenstein
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OELD
Barnhart
12/23/85

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P PDR

Mr. J. J. Carey
Duquesne Light Company

Beaver Valley 1 Power Station

cc:

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Duquesne Light Company

- 2 -

Beaver Valley 1 Power Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.99
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated June 17, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 99, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This amendment is effective on issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director
PWR Project Directorate #2

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 23, 1985

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 3-60	3/4 3-60
3/4 3-61	3/4 3-61
3/4 3-65	3/4 3-65
3/4 3-66	3/4 3-66
3/4 3-68	3/4 3-68

TABLE 3.3-13

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

	<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABILITY</u>	<u>PARAMETER</u>	<u>ACTION</u>
1.	Gaseous Waste/Process Vent System (RM-GW-108A & B)				
a.	Noble Gas Activity Monitor	(1)	*	Radioactivity Rate Measurement	27, 30 ***
b.	Particulate Activity Monitor	(1)	*		32
c.	System Effluent Flow Rate Measuring Device (FR-GW-108)	(1)	*	System Flow Rate Measurement	28
d.	Sampler Flow Rate Measuring Device	(1)	*	Sampler Flow Rate Measurement	28
2.	Auxiliary Building Ventilation System (RM-VS-101A & B)				
a.	Noble Gas Activity Monitor	(1)	*	Radioactivity Rate Measurement	29, 30 ***
b.	Particulate Activity Monitor	(1)	*		32
c.	System Effluent Flow Rate Measuring Device (FR-VS-101)	(1)	*	System Flow Rate Measurement	28
d.	Sampler Flow Rate Measuring Device	(1)	*	Sampler Flow Rate Measurement	28

* During Releases via this pathway

*** During purging of Reactor Containment via this pathway.

TABLE 3.3-13, (Continued)

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABILITY</u>	<u>PARAMETER</u>	<u>ACTION</u>
3. Reactor Building/Supplementary Leak Collection and Release System (RM-VS-107A & B)				
a. Noble Gas Activity Monitor	(1)	*	Radioactivity Rate	29, 30.***
b. Particulate Activity Monitor	(1)	*		32
c. System Effluent Flow Rate Measuring Device (FR-VS-112)	(1)	*	System Flow Rate Measurement	28
d. Sampler Flow Rate Measuring Device	(1)	*	Sampler Flow Rate Measurement	28

* During Releases via this pathway
 *** During purging of Reactor Containment via this pathway.

TABLE 4.3-13

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>SOURCE CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>
1. Gaseous Waste/Process Vent System (RM-GW-108A & B)				
a. Noble Gas Activity Monitor	P	P(5)	R(3)	Q(1)
b. Particulate Activity Monitor	W	N/A	N/A	N/A
c. System Effluent Flow Rate Measuring Device (FR-GW-108)	P	N/A	R	Q
d. Sampler Flow Rate Measuring Device	D*	N/A	R	Q
2. Auxiliary Building Ventilation System (RM-VS-101A&B)				
a. Noble Gas Activity Monitor	D	M(5), P(5)***	R(3)	Q(2)
b. Particulate Activity Monitor	W	N/A	N/A	N/A
c. System Effluent Flow Rate Measurement Device (FR-VS-101)	D	N/A	R	Q
d. Sampler Flow Rate	D	N/A	R	Q
3. Reactor Building/Supplementary Leak Collection and Release System (RM-VS-107A & B)				
a. Noble Gas Activity Monitor	D	M(5), P(5)***	R(3)	Q(2)

BEAVER VALLEY - UNIT 1

3/4 3-65

Amendment No. 99

TABLE 4.3-13, (Continued)

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

	<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>SOURCE CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>
b.	Particulate Activity Monitor	W	N/A	N/A	N/A
c.	System Effluent Flow Rate Measuring Device (FR-VS-112)	D	N/A	R	Q
d.	Sampler Flow Rate Measuring Device	D	N/A	R	Q
4.	Waste Gas Decay Tanks Monitor				
a.	Oxygen Monitor (O ₂ -AS-GW-110-1,2)	D	N/A	Q(4)	M
b.	Radiation Monitor (RM-GW-101)	D**	M(5)	R(3)	Q(2)
c.	Sampler Flow Rate Measuring Device	D**	N/A	R	Q

BEAVER VALLEY - UNIT 1

3/4 3-66

Amendment No. 99

TABLE 4.3-13 (Continued)

TABLE NOTATION

- (4) The CHANNEL CALIBRATION shall include the use of standard gas samples containing a nominal:
 1. One volume percent oxygen, balance nitrogen, and
 2. Four volume percent oxygen, balance nitrogen

- (5) A source check may be performed utilizing the installed means or flashing the detector with a portable source to obtain an upscale increase in the existing count rate to verify channel response.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

Introduction

By application dated June 17, 1985, Duquesne Light Company (the licensee), submitted a proposed amendment to the Technical Specification set forth in Appendix A to Operating License No. DPR-66 for the Beaver Valley Power Station, Unit 1. This amendment would remove the iodine sampler cartridges from the radioactive gaseous effluent monitoring instrumentation listed in Tables 3.3-13 and 4.3-13.

Discussion and Evaluation

During the implementation of NUREG-0737 (post-TMI) action items, the licensee became aware of the projected difficulties of retrieving and analyzing iodine sampler cartridges from the gaseous effluent monitors located in various buildings under accident conditions. The licensee has redundant capability for monitoring effluent releases: the original gaseous effluent monitor (Victoreen) and the more recent Special Particulate, Iodine and Noble Gas (SPING) radiation monitor (Eberline). Iodine sampler cartridges can be retrieved from the SPING system with less hazard to site personnel during emergencies. The licensee, therefore, requested a deletion of the requirement to maintain iodine sampler cartridges in the original gaseous effluent monitors.

The proposed change would not remove the requirement for iodine sampling, analysis and reporting. This would continue to be required under Section 4.11.2 (Radiological Environmental Technical Specification RETS), and the licensee has routinely used the SPING iodine cartridges as a source of data for the effluent reporting requirements in accordance with Regulatory Guide 1.21.

This proposal is consistent with the Beaver Valley Unit 1 FINAL SAFETY ANALYSIS Report Section 11.3.3.3, "Monitors", i.e., both the Gaseous Waste Gas Monitor (Section 11.3.3.3.2) and the High Range Noble Gas Monitors (Section 11.3.3.3.23) have provision for an iodine sampler cartridge but do not require their continuous use.

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The licensee performed a study in response to NRC Unresolved Item 50-334/83-30-05 to determine the capability of the SPING monitors to collect representative samples. The Unresolved Item resulted from a routine radiation safety inspection, in which the inspector questioned whether the wide separation between the sample nozzle location and the monitoring cabinets could result in losses in the interconnecting piping. This could result in erroneously low readings for radioactive particulates and iodine. The systems affected included Process Vent, Ventilation Vent and the Supplementary Leak Collection and Release System (SLCRS). Each is a potential release path to the environment for airborne activity.

On December 14, 1984, the Duquesne Light Company sent the results of an evaluation of the effluent radiation monitors to the Regional Administrator. The study carefully profiles the air flow in one section of the SLCRS and resulted in the design of a temporary isokinetic sampling probe. The licensee used guidance contained in ANSI N13.1 and ANSI N13.10 (Now ANSI N42.18-1980). Grab samples of iodine activity obtained via this probe for a six-month period were compared with readings from the permanently installed monitors. Good correlation was obtained for the iodine activity. The licensee's report has been reviewed (Inspection Report No. 50-334/85-17) and found to provide a reasonable and technically competent approach to the resolution of NRC Unresolved Item 83-30-05. The NRC concurred that good correlation was obtained among all radiation monitors for measuring Iodine 131.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not

be inimical to the common defense and security or to the health and safety of the public.

Dated: December 23, 1985

Principal Contributors:

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