Docket No. 50-334 October 9, 1984

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Mr. J. J. Carey, Vice President
Nuclear Division
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

Dear Mr. Carey:

The Commission has issued the enclosed Amendment No. 80 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application dated May 21, 1984.

The amendment changes the Technical Specifications for Beaver Valley Unit No. 1 to require either the sump discharge flow measurement system or the sump level instrument be operable during plant operation. Prior to this amendment, the specifications allowed no choices but simply required that the former be operable. The amendment also corrects a number of administrative errors in the Technical Specifications. Other changes requested in the same letter will be addressed by future actions.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

/s/PTam

Peter Tam, Project Manager
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:
1. Amendments No. 80 to DPR-66
2. Safety Evaluation

cc: w/enclosures
See next page

ORB#1:DL	ORB#1:DL	C-ORB#1:DL	OELD	AD:OR:DL
CParrish	PTam	SVarga		GLainas
9/25/84	9/26/84	9/26/84	10/3/84	10/9/84

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 9, 1984

Docket No. 50-334

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Nuclear Division
Duquesne Light Company
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Sincerely,

Peter S. Tam

Peter Tam, Project Manager
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:
1. Amendments No. 80 to DPR-66
2. Safety Evaluation

cc: w/enclosures
See next page

cc: Mr. W. S. Lacey
Station Superintendent
Duquesne Light Company
Beaver Valley Power Station
Post Office Box 4
Shippingport, PA 15007

Mr. K. Grada, Superintendent
of Licensing and Compliance
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

Mr. John A. Levin
Public Utility Commission
Post Office Box 3265
Harrisburg, PA 17120

Gerald Charnoff, Esquire
Jay E. Silberg, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, DC 20036

Karin Carter, Esquire
Special Assistant Attorney General
Bureau of Administrative Enforcement
5th Floor, Executive House
Harrisburg, PA 17120

Marvin Fein
Utility Counsel
City of Pittsburgh
313 City-County Building
Pittsburg, PA 15219

Resident Inspector
U.S. Nuclear Regulatory Commission
Post Office Box 298
Shippingport, PA 15077

Department of Environmental Resources
ATTN: Director, Office of Radiological Health
Post Office Box 2063
Harrisburg, PA 17105

Mr. Thomas J. Czerpah
Mayor of the Burrough of
Shippingport
Post Office Box 26
Shippingport, PA 15077

Pennsylvania Power Company
Ray E. Sempler
One E. Washington Street
New Castle, PA 16103

Ohio Environmental Protection Agency
Division of Planning
Environmental Assessment Section
Post Office Box 1049
Columbus, Ohio 43216

Office of the Governor
State of West Virginia
Charleston, West Virginia 25305

Charles A. Thomas, Esquire
Thomas and Thomas
212 Locust Street
Box 999
Harrisburg, PA 17108

Regional Radiation Representative
EPA Region III
Curtis Building - 6th Floor
Philadelphia, PA 19106

Governor's Office of State Planning
and Development
ATTN: Coordinator, Pennsylvania
State Clearinghouse
Post Office Box 1323
Harrisburg, PA 17120

Mr. Joseph H. Mills, Acting Commissioner
State of West Virginia Department
of Labor
1900 Washington Street
East Charleston, West Virginia 25305

cc: N. H. Dyer, M.D.
State Director of Health
State Department of Health
1800 Washington Street, East
Charleston, West Virginia 25305

Irwin A. Popowsky, Esquire
Office of Consumer Advocate
1425 Strawberry Square
Harrisburg, PA 17120

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 80
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated May 21, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 80 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This amendment is effective on issuance, to be implemented no more than 30 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 9, 1984

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 80 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 3-56	3/4 3-56
3/4 4-4	3/4 4-4
3/4 4-11	3/4 4-11
3/4 4-12	3/4 4-12
3/4 7-14	3/4 7-14
5-1b	5-1b

TABLE 3.3-12 (Continued)

TABLE NOTATION

ACTION 23 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases may be resumed provided that prior to initiating a release:

1. At least two independent samples are analyzed in accordance with specification 4.11.1.1.1, and;

2. At least two technically qualified members of the Facility Staff independently verify the release rate calculations and discharge valving;

Otherwise, suspend release of radioactive effluents via this pathway.

ACTION 24 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided that at least once per 8 hours grab samples are analyzed for gross radioactivity (beta or gamma) at a Lower Limit of Detection (LLD) of at least 10^{-7} μCi/ml.

ACTION 25 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue provided the flow rate is estimated at least once per 4 hours during actual releases. Pump curves may be used to estimate flow.

ACTION 26 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, liquid additions to this tank may continue provided the tank liquid level is estimated during all liquid additions to the tank.

ISOLATED LOOP STARTUP

LIMITING CONDITION FOR OPERATION

3.4.1.5 A reactor coolant loop shall remain isolated until:

a. The isolated loop has been operating on a recirculation flow of $\geq$125 gpm for at least 90 minutes and the temperature at the cold leg of the isolated loop is within 20°F of the highest cold leg temperature of the operating loops.

b. The reactor is subcritical by at least 1 percent $\Delta k/k$.

APPLICABILITY: ALL MODES.*

ACTION:

With the requirements of the above specification not satisfied, suspend startup of the isolated loop.

SURVEILLANCE REQUIREMENTS

4.4.1.5.1 The isolated loop cold leg temperature shall be determined to be within 20°F of the highest cold leg temperature of the operating loops within 30 minutes prior to opening the cold leg stop valve.

4.4.1.5.2 The reactor shall be determined to be subcritical by at least 1 percent $\Delta k/k$ within 30 minutes prior to opening the cold leg stop valve.

* With fuel in the vessel.

3/4.4.6 REACTOR COOLANT SYSTEM LEAKAGE

LEAKAGE DETECTION SYSTEMS

LIMITING CONDITION FOR OPERATION

3.4.6.1 The following Reactor Coolant System leakage detection systems shall be OPERABLE:

a. The containment atmosphere particulate radioactivity monitoring system,

b. The containment sump discharge flow measurement system or narrow range level instrument, and

c. Containment atmosphere gaseous radioactivity monitoring system.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

a. With one of the above required radioactivity monitoring leakage detection systems inoperable, operations may continue for up to 30 days provided:

   1. The other two above required leakage detection systems are OPERABLE, and

   2. Appropriate grab samples are obtained and analyzed at least once per 24 hours:

   otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

b. With the containment sump discharge flow measurement system and narrow range level instrument inoperable, restore at least one inoperable system to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

c. The provisions of specification 3.0.4 are not applicable in Modes 1, 2 and 3.

SURVEILLANCE REQUIREMENTS

4.4.6.1 The leakage detection systems shall be demonstrated OPERABLE by:

a. Containment atmosphere particulate and gaseous monitoring system-performance of CHANNEL CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST at the frequencies specified in Table 4.3-3,

SURVEILLANCE REQUIREMENTS (Continued)

b. Containment sump discharge flow measurement system-performance of CHANNEL CALIBRATION TEST at least once per 18 months.

c. Logging the narrow range level indication every 12 hours.

PLANT SYSTEMS

3/4.7.5 ULTIMATE HEAT SINK - OHIO RIVER

LIMITING CONDITION FOR OPERATION

3.7.5.1 The ultimate heat sink shall be OPERABLE with:

a. A minimum water level at or above elevation 654 Mean Sea Level, at the intake structure, and

b. An average water temperature of $\leq 86°F$.

APPLICABILITY: MODES 1, 2, 3 and 4.

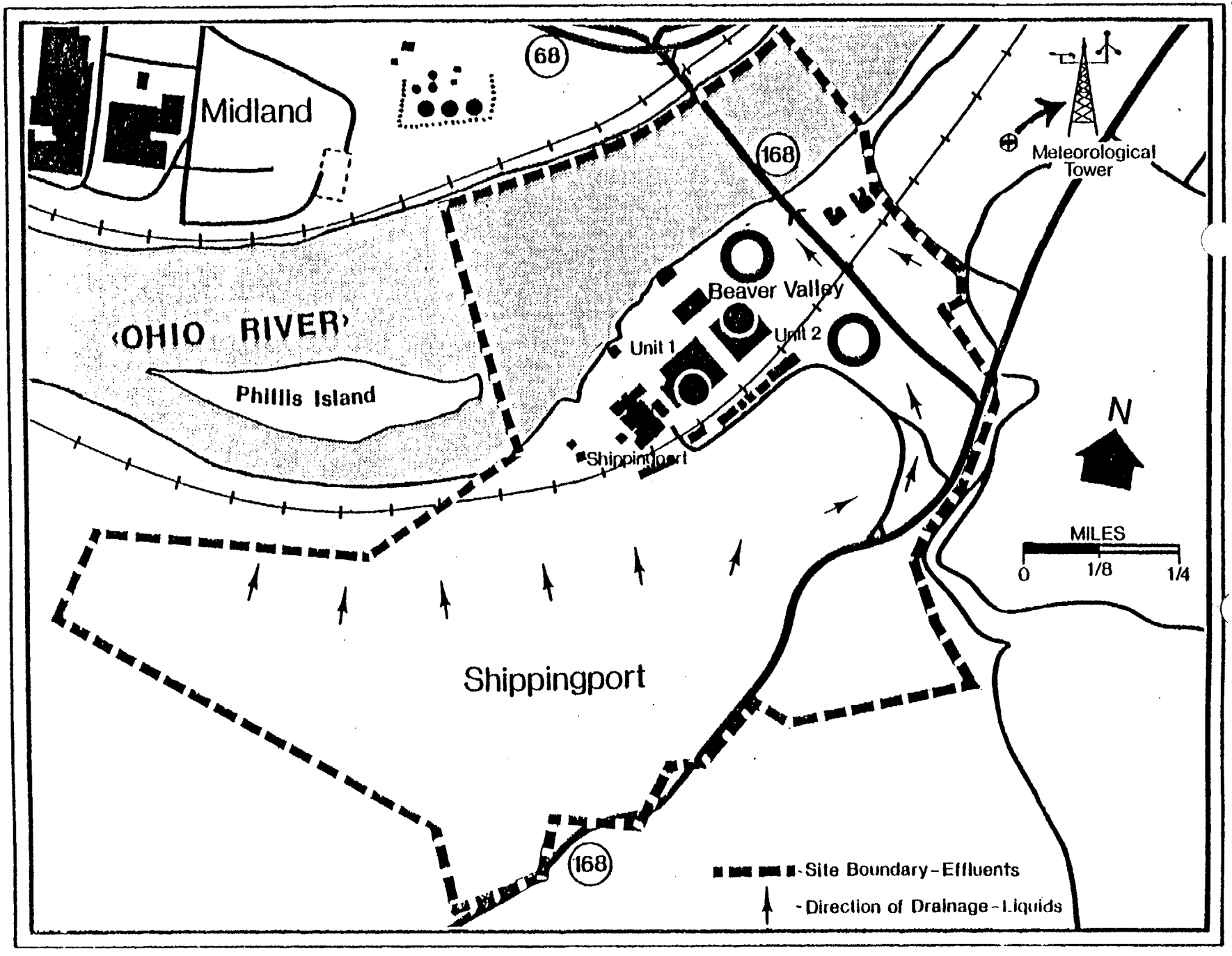
ACTION:

With the requirements of the above specification not satisfied, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.5.1 The ultimate heat sink shall be determined OPERABLE at least once per 24 hours by verifying the average water temperature and water level to be within their limits.

SITE BOUNDARY FOR GASEOUS EFFLUENTS
FOR THE BEAVER VALLEY POWER STATION



BEAVER VALLEY - UNIT 1 FIGURE 5.1-1

5-1b

Amendment No. 80

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 80 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

Introduction

By letter dated May 21, 1984, Duquesne Light Company (the licensee) requested a number of changes to the Technical Specifications appended to Operating License No. DPR-66 for Beaver Valley Unit 1. The staff has completed review of a few of these changes and the results are described as follows.

Evaluation and Discussion

The licensee requested that new technical specifications be imposed on the narrow range containment sump level instrument. In the meantime, the specifications are changed such that either the sump level instrument or sump discharge flow measurement system need be operable during normal operation. The changes are:

1. Adding the narrow range sump level instrument in the limiting condition for operation in Section 3.4.6.1, as per Updated Final Safety Analysis Report (UFSAR) Section 7.3.1.3.1. The narrow range sump level instrument provides an additional method of leakage indication and monitors the sump level under normal operating conditions.

2. Narrow range level instrument operability is added to the Action Statement B in Section 3.4.6.1 as follows:

   "With the containment sump discharge flow measurement system and narrow range level instrument inoperable, restore at least one inoperable system to OPERABLE status within seven days or be in at least HOT STANDBY within the next six hours and in COLD SHUTDOWN within the following 30 hours."

3. Adding the Action Statement C in Section 3.4.6.1, to permit the plant to be restarted following a plant trip. Alternate and diverse methods of monitoring reactor coolant system (RCS) leakage are available: containment sump discharge flow measurement or narrow range sump level. Therefore, should one of these systems become inoperable, the other system would be available to monitor and detect RCS leakage.

4. Surveillance requirements for logging the narrow range level indication every 12 hours is included in Section 4.4.6.1.

The above four changes to the technical specifications will have the effect of requiring either the sump level instrument or the sump discharge flow measurement system be operable during normal operation. Previously, only the latter was required. Either is capable of detecting reactor coolant leakage and would provide prompt notification to the operators. All components and systems are already in place and in use; no hardware change is involved with this amendment. We conclude that the requested changes are acceptable.

The licensee also requested a number of changes to the Technical Specifications to correct administrative errors due to previous amendments. These corrections are all acceptable and are issued as amended Pages 3/4 3-56, 3/4 7-14 and 5-1b of the Technical Specifications.

The licensee further requested that a footnote be added to Specification 3.4.1.5, "Isolated Loop Startup". The footnote would specify that the specification is applicable only when fuel is present in the reactor vessel. This specification is intended to prevent a reactivity transient due to the injection of cool water from the startup of an idle loop. Therefore, with no fuel in the vessel, there can be no reactivity transient and no need for this specification. The requested change merely clarifies the specification and is acceptable to the staff.

Environmental Consideration

This amendment involves only changes in administrative procedure and requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 9, 1984

Principal Contributor:

R. Goel