DISTRIBUITON
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NRC PDR
Local PDR
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MAR 22 1983

Docket No. 50-334

Mr. J. J. Carey, Vice President
Duquesne Light Company
Nuclear Division
Post Office Box 4
Shippingport, Pennsylvania 15077

Dear Mr. Carey:

The Commission has issued the enclosed Amendment No. 65 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated August 3, 1982, supplemented by letters dated October 5 and October 25, 1982.

This amendment changes the Technical Specifications to waive the need of Type-C testing for certain valves. These valves do not represent potential containment atmosphere leakage paths and, therefore, need not be subjected to the provisions of Appendix J, Section C. Most of these valves have already been included in your Inservice Testing Program (IST), those that are not should be considered for inclusion. Valve liquid leakage limits should be specified in your IST program, but these limits need not be incorporated into your Technical Specifications.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

/S/

Peter S. Tam, Project Manager
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:
1. Amendment No. 65 to DPR-66
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

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				no legal objection			
OFFICE	ORB 1	ORB 1	ORB 1	AD:OR:DL	OELD		
SURNAME	CParrish/	PTam/rs	SVarga	GLainas			
DATE	3/16/83	3/18/83	3/1/83	3/14/83	3/15/83		

Mr. J. J. Carey
Duquesne Light Company

cc: Mr. W. S. Lacey
Station Superintendent
Duquesne Light Company
Beaver Valley Power Station
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Mr. K. Grada, Superintendent
of Licensing and Compliance
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Utility Counsel
City of Pittsburgh
313 City-County Building
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Resident Inspector
U. S. Nuclear Regulatory Commission
Post Office Box 298
Shippingport, Pennsylvania 15077

Department of Environmental
Resources
ATTN: Director, Office of
Radiological Health
Post Office Box 2063
Harrisburg, Pennsylvania 17105

Mr. Thomas J. Czerpah
Mayor of the Burrough of
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Ohio Environmental Protection Agency
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Charleston, West Virginia 25305

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Thomas and Thomas
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Box 999
Harrisburg, West Virginia 17108

Mr. J. J. Carey
Duquesne Light Company

cc: Regional Radiation Representatives
EPA Region III
Curtis Building - 6th Floor
Philadelphia, Pennsylvania 19106

Governor's Office of State Planning
and Development
ATTN: Coordinator, Pennsylvania
State Clearinghouse
P.O. Box 1323
Harrisburg, Pennsylvania 17120

Mr. Joseph H. Mills, Acting Commissioner
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 65
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated August 3, 1982, supplemented by letters dated October 5 and October 25, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 65 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance:

March 22, 1983

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 65 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 6-3	3/4 6-3
3/4 6-19a	3/4 6-19a
3/4 6-19b	3/4 6-19b
3/4 6-19c	3/4 6-19c
3/4 6-19d	3/4 6-19d
3/4 6-19e	3/4 6-19e
3/4 6-19f	3/4 6-19f
-	3/4 6-19g
-	3/4 6-19h
-	3/4 6-19i
-	3/4 6-19j
-	3/4 6-19k

SURVEILLANCE REQUIREMENTS (Continued)

b. If any periodic Type A test fails to meet .75 L_a, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet .75 L_a, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet .75 L_a at which time the above test schedule may be resumed.

c. The accuracy of each Type A test shall be verified by a supplemental test which:

   1. Confirms the accuracy of the Type A test by verifying that the difference between supplemental and Type A test data is within 0.25 L_a.

   2. Has a duration sufficient to accurately establish the change in leakage for between the Type A test and the supplemental test.

   3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage rate at P_a (38.3 psig).

d. Type B and C tests shall be conducted with gas at P_a*(38.3 psig) at intervals no greater than 24 months except for tests involving:

   1. Air locks,

   2. Penetrations using continuous leakage monitoring systems, and

   3. Valves pressurized with fluid from a seal system.

e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.

f. Leakage from isolation valves that are sealed with fluid from a seal system may be excluded, subject to the provisions of Appendix J, Section III.C.3, when determining the combined leakage rate provided the seal system and valves are pressurized to at least 1.10 P_a (42.1 psig) and the seal system capacity is adequate to maintain system pressure for at least 30 days.

* Applicable valves may be tested using water as the pressure fluid in accordance with the Inservice Testing Program.

TABLE 3.6-1
CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME*(SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME*(SEC)
1-D	CCR to RHS Hx 1A & RHS Pump 1A Seal Cooler	(1)MOV-1CC-112A2	N/A	(1)1CCR-247	N/A
2-D	CCR from RHS Hx 1B & RHS Pump 1B Seal Cooler	(1)MOV-1CC-112B3	N/A	(1)1CCR-252	N/A
3	Spare				
4-D	CCR from RHS Hx 1A & RHS Pump 1A Seal Cooler	(1)MOV-1CC-112A3	N/A	(1)1CCR-251	N/A
5-D	CCR to RHS Hx 1B & RHS Pump 1B Seal Cooler	(1)MOV-1CC-112B2	N/A	(1)1CCR-248	N/A
6-B	Spare				
7-A	High Head SI to Hot Legs	(2)1SI-83	N/A	(2)MOV-1SI-869A	N/A
8-C	CCR from RCP 1B & 1C Thermal Barriers	(B)TV-1CC-107D1	20	(B)TV-1CC-107D2	20
9-B	CCR from Shroud Coolers	(B)TV-1CC-111D1	20	(B)TV-1CC-111D2	20
10-B	Spare				
11-B	Air Recirc. Cooling Water-Out	(B)TV-1CC-110D	30	(B)TV-1CC-110F2 (B)TV-1CC-110F1	30 30
12-A	Spare				
13-D	Deluge System to CNMT Hose Reels	1FP-827	N/A	TV-1FP-107	N/A
14-D	Air Recirc. Cooling Water-In	(B)TV-1CC-110E3	30	(B)TV-1CC-110E2	30
15-A	Coolant System Charging	(2)1CH-31	N/A	(2)MOV-1CH-289	15
16-B	CCR to Shroud Coolers	(B)TV-1CC-111A2	20	(B)TV-1CC-111A1	20

TABLE 3.6-1
CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME*(SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME*(SEC)
17-A	CCR to RCP 1B	(B)TV-1CC-103B1	20	(B)TV-1CC-103B	20
18-A	CCR to RCP 1C	(B)TV-1CC-103C1	20	(B)TV-1CC-103C	20
19-A	RCP's Seal Water Return	(A)MOV-1CH-378 1CH-369	15 N/A	(A)MOV-1CH-381	1
20-C	SI Accum. Makeup	1SI-42	N/A	(1)1SI-41	N/A
21-B	Spare				
22-B	Spare				
23-B	Spare				
24-SgD	RHS to RWST	1RH-14 1RH-16	N/A N/A	1RH-15	N/A
25-B	CCR from RCP 1B & 1C Motors	(B)TV-1CC-105D1	20	(B)TV-1CC-105D2	20
26-C	CCR from RCP 1A Thermal Barrier	(B)TV-1CC-107E1	10	(B)TV-1CC-107E2	10
27-C	CCR from RCP 1A Motor	(B)TV-1CC-105E1	14	(B)TV-1CC-105E2	14
28-A	RCS Letdown	(A)TV-1CH-200A (A)TV-1CH-200B (A)TV-1CH-200C (1)MOV-1CH-142 RV-1CH-203	7.5 7.5 7.5 N/A N/A	(A)TV-1CH-204	7.5
29-A	Primary Drain Transfer Pump #1 Discharge	(A)TV-1DG-108A	5	(A)TV-1DG-108B	5
30-B	Spare				
31-D	Deluge System to Cable Penetration Area	1FP-804	N/A	TV-1FP-105	N/A

TABLE 3.6-1
CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME*(SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME*(SEC)
32-C	Deluge System to RHR Area	1FP-800	N/A	TV-1FP-106	N/A
33-C	High Head SI to Hot Legs	(2)1SI-84	N/A	(2)MOV-1SI-869B	N/A
34-A	Spare				
35-A	Seal Injection Water RCP 1A	(2)1CH-181	N/A	(2)MOV-1CH-308A	N/A
36-A	Seal Injection Water RCP 1B	(2)1CH-182	N/A	(2)MOV-1CH-308B	N/A
37-A	Seal Injection Water RCP 1C	(2)1CH-183	N/A	(2)MOV-1CH-308C	N/A
38-A	Containment Sump Pump Discharge	(A)TV-1DA-100A	10	(A)TV-1DA-100B	10
39-C	Steam Generator 1A Blowdown	Closed System	N/A	(2)(A)TV-1BD-100A	20
40-A	Steam Generator 1B Blowdown	Closed System	N/A	(2)(A)TV-1BD-100B	20
41-B	Steam Generator 1C Blowdown	Closed System	N/A	(2)(A)TV-1BD-100C	20
42-C	Compressed Air to Fuel Handling Equipment	1SA-15	N/A	1SA-14	N/A
43-B	Air Activity Monitor-Out	(A)TV-1CV-102-1	5	(A)TV-1CV-102	5
44-B	Air Activity Monitor-In			(A)TV-1CV-101A (A)TV-1CV-101B	5 5
45-B	Primary Grade Water to PRT	1RC-72	N/A	(A)TV-1RC-519	12
46-A	Charging Fill Header	(2)1CH-170	N/A	(2)(1)FCV-1CH-160	N/A
47-B	Instrument Air	1IA-91	N/A	1IA-90	N/A

TABLE 3.6-1
CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME*(SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME*(SEC)
48-B	Primary Vent Header	(A)TV-1DG-109A2	5	(A)TV-1DG-109A1	5
49-C	Nitrogen Supply to PRT	1RC-68	N/A	(A)TV-1RC-101	5
50-C	Spare				
51-C	Spare				
52-C	Spare				
53-C	Nitrogen Supply to SI Accumulators	(A)TV-1SI-101-2	5	(A)TV-1SI-101-1	5
54-B	Spare				
55-1-A	SI Accumulator Sample	(A)TV-1SS-109A1	20	(A)TV-1SS-109A2	20
55-2-A	CNMT Leakage Monitoring Open Taps			(A)TV-1LM-100A1 (A)TV-1LM-100A2	5 5
55-3-A	Spare				
55-4-A	PRT Gas Sample	(A)TV-1SS-111A1	20	(A)TV-1SS-111A2	20
56-1-A	Pressurizer Liquid Sample	(A)TV-1SS-100A1	20	(A)TV-1SS-100A2	20
56-2-A	RCS Cold Leg Sample	(A)TV-1SS-102A1	20	(A)TV-1SS-102A2	20
56-3-A	RCS Hot Leg Samples	(A)TV-1SS-105A1	20	(A)TV-1SS-105A2	20
56-4-A	STM GEN 1A Blowdown Sample	Closed System	N/A	(2)(A)TV-1SS-117A	20
57-1-A	CNMT Leakage Monitoring Open Taps			(A)TV-1LM-100A1 (A)TV-1LM-100A2	5 5

TABLE 3.6-1
CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME*(SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME*(SEC)
57-2-A	CNMT Leakage Monitoring Open Taps			(A)TV-1LM-100A1 (A)TV-1LM-100A2	5 5
57-3-A	CNMT Leakage Monitoring Sealed System	(A)TV-1LM-101A	5	1CV-35	N/A
57-4-A	CNMT Leakage Monitoring Sealed System	(A)TV-1LM-101B	5	1CV-36	N/A
58-B	CCR to RCP 1A	(B)TV-1CC-103A1	20	(B)TV-1CC-103A	20
59-C	Spare				
60-SgD	Low Head SI to Hot Legs	(2)1SI-13	N/A	(2)MOV-1SI-890A (2)1SI-451	N/A N/A
61-SgD	Low Head SI to Cold Legs	(2)1SI-10 (2)1SI-11 (2)1SI-12	N/A N/A N/A	(2)MOV-1SI-890C	N/A
62-SgD	Low Head SI to Hot Legs	(2)1SI-14	N/A	(2)MOV-1SI-890B (2)1SI-452	N/A N/A
63-SgD	QSP Discharge 360° Header	1QS-4	N/A	(B)MOV-1QS-101B	75(4)
64-SgD	QSP Discharge 360° Header	1QS-3	N/A	(B)MOV-1QS-101A	75(4)
65	Fuel Transfer Tube	(7) Flange	N/A		
66-SgD	Outside RSP 2A Suction from CNMT			(B)(2)MOV-1RS-155A	75(4)
67-SgD	Outside RSP 2B Suction from CNMT			(B)(2)MOV-1RS-155B	75(4)
68-SgD	Low Head SI Pump 1A Suction from CNMT Sump			(9)(2)MOV-1SI-860A	N/A
69-SgD	Low Head SI Pump 1B Suction from CNMT Sump			(9)(2)MOV-1SI-860B	N/A

TABLE 3.6-1
CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME*(SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME*(SEC)
70-SgD	Outside RSP 2B Discharge	1RS-101	N/A	(B)(2) MOV-1RS-156B	75(4)
71-SgD	Outside RSP 2A Discharge	1RS-100	N/A	(B)(2) MOV-1RS-156A	75(4)
72-SgD	Spare				
73-SgD	Main Steam Loop 1A	Closed System	N/A	(2)TV-1MS-101A	5
	Main Steam Line Drain	Closed System	N/A	(2)TV-1MS-111A	10
	Main Steam to Auxiliary Feed Pump	Closed System	N/A	(2)MOV-1MS-105	N/A
	Main Steam Atmospheric Dump	Closed System	N/A	(6)PCV-1MS-101A	N/A
	Main Steam Safety Valves	Closed System	N/A	(6)Safety Valves	N/A
74-SgD	Main Steam Loop 1B	Closed System	N/A	(2)TV-1MS-101B	5
	Main Steam Line Drain	Closed System	N/A	(2)TV-1MS-111B	10
	Main Steam to Auxiliary Feed Pump	Closed System	N/A	(2)MOV-1MS-105	N/A
	Main Steam Atmospheric Dump	Closed System	N/A	(6)PCV-1MS-101B	N/A
	Main Steam Safety Valves	Closed System	N/A	(6)Safety Valves	N/A
75-SgD	Main Steam Loop 1C	Closed System	N/A	(2)TV-1MS-101C	5
	Main Steam Line Drain	Closed System	N/A	(2)TV-1MS-111C	10
	Main Steam to Auxiliary Feed Pump	Closed System	N/A	(2)MOV-1MS-105	N/A
	Main Steam Atmospheric Dump	Closed System	N/A	(6)PCV-1MS-101C	N/A
	Main Steam Safety Valves	Closed System	N/A	(6)Safety Valves	N/A
76-SgD	FW Loop 1A	Closed System	N/A	(2)MOV-1FW-156A	75
	FW Loop 1A	Closed System	N/A	(2)FCV-1FW-478	10
	FW Loop 1A	Closed System	N/A	(6)MOV-1FW-158A	N/A
	Auxiliary Feedwater Loop 1A	Closed System	N/A	(2)MOV-1FW-151E	N/A
	Auxiliary Feedwater Loop 1A	Closed System	N/A	(2)MOV-1FW-151F	N/A
77-SgD	FW Loop 1B	Closed System	N/A	(2)MOV-1FW-156B	75
	FW Loop 1B	Closed System	N/A	(2)FCV-1FW-488	10
	FW Loop 1B	Closed System	N/A	(6)MOV-1FW-158B	N/A
	Auxiliary Feedwater Loop 1B	Closed System	N/A	(2)MOV-1FW-151C	N/A
	Auxiliary Feedwater Loop 1B	Closed System	N/A	(2)MOV-1FW-151D	N/A

TABLE 3.6-1
CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME*(SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME*(SEC)
78-SgD	FW Loop 1C	Closed System	N/A	(2)MOV-1FW-156C	75
	FW Loop 1C	Closed System	N/A	(2)FCV-1FW-498	10
	FW Loop 1C	Closed System	N/A	(6)MOV-1FW-158C	N/A
	Auxiliary Feedwater Loop 1C	Closed System	N/A	(2)MOV-1FW-151A	N/A
	Auxiliary Feedwater Loop 1C	Closed System	N/A	(2)MOV-1FW-151B	N/A
79-SgD	RW to 1A RSP Hx	Closed System	N/A	(2)MOV-1RW-104A	N/A
80-SgD	RW to 1C RSP Hx	Closed System	N/A	(2)MOV-1RW-104C	N/A
81-SgD	RW to 1B RSP Hx	Closed System	N/A	(2)MOV-1RW-104B	N/A
82-SgD	RW to 1D RSP Hx	Closed System	N/A	(2)MOV-1RW-104D	N/A
83-SgD	RW from 1A RSP Hx	Closed System	N/A	(2)MOV-1RW-105A	N/A
84-SgD	RW from 1C RSP Hx	Closed System	N/A	(2)MOV-1RW-105C	N/A
85-SgD	RW from 1B RSP Hx	Closed System	N/A	(2)MOV-1RW-105B	N/A
86-SgD	RW from 1D RSP Hx	Closed System	N/A	(2)MOV-1RW-105D	N/A
87-SgD	H_2 Discharge to CNMT	1HY-120	N/A	1HY-111	N/A
88-SgD	H_2 Discharge to CNMT	1HY-119	N/A	1HY-110	N/A
89-SgD	Main Condenser Ejector Vent	1AS-278	N/A	(B)TV-1SV-100A	20
90-SgD	CNMT Purge Exhaust	VS-D-5-3B	(5)8	VS-D-5-3A	(5)8
91-SgD	CNMT Purge Supply	VS-D-5-5B	(5)11	VS-D-5-5A	(5)8
				VS-D-5-6	N/A

TABLE 3.6-1
CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME*(SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME*(SEC)
92-A	CNMT Vacuum Pump 1B & H_2 Recomb. Suction			(A)TV-1CV-150C	7.5
				(A)TV-1CV-150D	7.5
				1HY-102	N/A
				1HY-104	N/A
93-B	CNMT Vacuum Pump 1A & H_2 Recomb. Suction			(A)TV-1CV-150A	7.5
				(A)TV-1CV-150B	7.5
				1HY-101	N/A
				1HY-103	N/A
94-C	CNMT Vacuum Ejector Suction	HCV-1CV-151	N/A	HCV-1CV-151-1	N/A
95-64	H_2 Analyzer - CNMT Dome	SOV-1HY-102B1	N/A	SOV-1HY-102B2	N/A
95-69	H_2 Analyzer - PRZR Cubicle	SOV-1HY-103B1	N/A	SOV-1HY-103B2	N/A
95-72	H_2 Analyzer Discharge	SOV-1HY-104B1	N/A	SOV-1HY-104B2	N/A
96-B	High Head SI to Cold Legs	(2)1SI-95	N/A	(2)MOV-1SI-836	N/A
97-1-A	RHR Inlet Sample	(A)TV-1SS-104A1	20	(A)TV-1SS-104A2	20
97-2-A	RHR Outlet Sample	(A)TV-1SS-103A1	20	(A)TV-1SS-103A2	20
97-3-A	CNMT Leakage Monitoring Open Taps			(A)TV-1LM-100A1	5
				(A)TV-1LM-100A2	5
97-4-A	Steam Generator 1C Blowdown Sample	Closed System	N/A	(2)(A)TV-1SS-117C	20
98-1-C	Spare				

TABLE 3.6-1
CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME*(SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME*(SEC)
98-2-C	Spare				
98-3-C	Spare				
98-4-C	Spare				
99-C	Spare				
100-B	Spare				
101-B	Spare				
102-B	Spare				
103-A	Refueling Cavity Purification Inlet	1PC-38	N/A	1PC-37	N/A
104-A	Refueling Cavity Purification Outlet	1PC-9	N/A	1PC-10	N/A
105-1-B	Steam Generator 1B Blowdown Sample	Closed System	N/A	(2)(A)TV-1SS-117B	20
105-2-B	PRZR Vapor Sample	(A)TV-1SS-112A1	20	(A)TV-1SS-112A2	20
105-3-B	Spare				
105-4-B	Spare				
106-SgD	SI Accumulator Test Line	(A)MOV-1SI-842	15	(A)TV-1SI-889	7.5
107-C	Spare				
108-B	Spare				
109-44	Inlet Flow Sample - CNMT Dome	SOV-1HY-102A1	N/A	SOV-1HY-102A2	N/A
109-49	Inlet Flow Sample - PRZR Cubicle	SOV-1HY-103A1	N/A	SOV-1HY-103A2	N/A
109-52	Flow Sample Discharge	SOV-1HY-104A1	N/A	SOV-1HY-104A2	N/A

TABLE 3.6-1
CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME*(SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME*(SEC)
110-1-C	PRZR Dead Weight Calibrator PT-RC-455A	Closed System	N/A	(1)1RC-277	N/A
				(1)1RC-278	N/A
110-2-C	Spare				
110-3-C	Spare				
110-4-C	Spare				
111-C	Spare			(7) Flange	N/A
112-C	Spare			(7) Flange	N/A
113-1-A	BIT to Cold Legs	(2)1SI-94	N/A	(2)MOV-1SI-867C	15
				(2)MOV-1SI-867D	15
				(3)(2)1SI-91	N/A
Primary Containment Airlock PH-P-1					
	Equalization Valve	(8)(7) SOV-1VS-2	N/A		
	Equalization Valve	(7)1VS-169	N/A		
	Equalization Valve	(7)1VS-170	N/A		
	Equalization Valve	(8)(7)SOV-1VS-4	N/A		
	Equalization Valve			(8)(7) SOV-1VS-5	N/A
	Equalization Valve			(7)1VS-167	N/A
	Equalization Valve			(7)1VS-168	N/A
	Equalization Valve			(8)(7)SOV-1VS-6	N/A

TABLE 3.6-1
CONTAINMENT PENETRATIONS

PENT. NO.-AREA	IDENTIFICATION/DESCRIPTION	INSIDE VALVE	MAXIMUM STROKE TIME*(SEC)	OUTSIDE VALVE	MAXIMUM STROKE TIME*(SEC)
Emergency Containment Airlock PH-P-2					
	Equalization Valve	(8)(7)1VS-172	N/A		
	Equalization Valve			(8)(7) 1VS-171	N/A

(A) Containment Isolation Phase A.
(B) Containment Isolation Phase B.

(1) May be opened on an intermittent basis under administrative control.

(2) Not subject to Type C leakage tests.

(3) May be leakage tested with water as the test fluid.

(4) Maximum opening time.

(5) Applicability: During CORE ALTERATIONS or movement of irradiated fuel within containment. The provisions of specification 3.0.4 are not applicable. The containment Purge Exhaust and Supply valves will be locked shut during operation in modes 1, 2, 3, and 4.

(6) Not subject to the requirements of specification 3/4.6.3. Listed in TABLE 3.6-1 for information only.

(7) Tested under Type "B" testing.

(8) Temporarily removed and penetration plugged.

(9) Auto open on Safety Injection recirculation signal.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 65 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

INTRODUCTION

By letter dated August 3, 1982 (Reference 1), the licensees proposed certain revisions to the Technical Specifications concerning leakage testing of certain containment isolation valves for Beaver Valley Power Station, Unit 1 (BVPS). Further clarification was provided in References 2 and 3 in response to a staff request. The proposed changes to the Technical Specifications involve revising Section 4.6.1.2d and Table 3.6-1 to indicate that certain valves may be leakage-tested using water as the pressure fluid rather than gas, as is currently required. A total of 34 valves in 16 various safety injection system lines have been proposed for water testing. In addition, the licensee has proposed that the requirement to Type-C-test 6 other valves on the secondary side of the steam generator be waived since these valves are located in closed systems. A list of the valves affected by the proposed changes to the Technical Specifications is presented in Table 1.

EVALUATION

Testing with water is consistent with the provisions of Appendix J to 10 CFR Part 50 and the NRC Standard Review Plan (NUREG-0800) Section 6.2.6, Containment Leakage Testing, which indicate that hydrostatic testing of containment isolation valves is permissable if the line is not a potential containment atmosphere leak path, and if a liquid inventory is available to maintain a water seal during the post-accident period (while assuming a single failure of any active component). The application of this approach to the subject isolation valves is somewhat unique because the BVPS uses a subatmospheric containment design.

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BVPS containment is essentially a dry containment maintained at a subatmospheric pressure during normal operation. Following a design-basis accident, the containment pressure rises above atmospheric with outleakage being possible, but returns to subatmospheric pressure within one hour through the use of a containment depressurization system. Outleakage of containment atmosphere through the isolation valves can occur only during the first hour since this is the only time at which the pressure differential between the containment and environment is positive. The requirements prescribed in Appendix J regarding water testing can still be applied, however, through appropriate consideration of single active failures in systems, and line pressures and/or water inventories. This is consistent with Appendix J Section IV.B.

We have reviewed the licensees' proposed changes in light of single active failures in the associated systems, and conclude that the use of water instead of gas as a test fluid is acceptable for all of the proposed valves except those in the quench spray system. The bases for acceptability are that the subject valves are:

1. Located in a discharge line of the low- or high-head safety injection pump and open with flow through the valves, or closed with a pressure head acting inward, in both the injection and recirculation modes of operation; or

2. Located in the suction line to the Low Head Safety Injection Pump and water-sealed by the sump water inventory following a LOCA.

The potential for outleakage during the first hour following onset of a LOCA is considered remote because the various safety systems in which the subject valves are located are expected to be operating during the one-hour period. With the exception of the quench spray system, these lines contain cross-connections between redundant trains and thus will be pressurized to well in excess of the peak containment pressure, even in the event of a single active failure. Furthermore, these safety-grade fluid lines are expected to remain water filled for the remainder of the 30-day period, thus providing a source of sealing water for this period. We conclude that these valves do not represent a containment atmosphere leakage path and, therefore, should not be subject to the provisions of Appendix J, Section III.C. However, the licensee should address the need to include valves in his Inservice Testing Program for valve leakage (e.g. valves 1SI-451 and 1SI-452).

As noted above, the quench spray pump discharge piping does not contain a cross-connection. In the event of a system train failure, pump discharge pressure cannot be assured at the inboard side of the corresponding isolation valve. Therefore, water-testing of the two quench spray system isolation valves (MOV-1QS-101A and MOV-1QS-101B) is permissible only if it can be demonstrated by test that the minimum water inventory in each of the fluid lines is sufficient to maintain a water-filled line at the isolation valves when subjected to a pressure of 1.1 times the calculated peak containment internal pressure.

In view of these requirements, the licensee has given further consideration to the capability to maintain a water seal at the quench spray isolation valves. The licensee has concluded that a water seal cannot be assured since an operator action is required to terminate spray injection and close the isolation valves. Accordingly, the licensee has dropped his request to test these valves with water, and will continue to Type-C-test the valves with air.

The licensee has also proposed that Type C leakage testing requirements be waived for six valves on the secondary side of the steam generator because they are located within a closed system and would not be exposed to the containment atmosphere. We have reviewed the containment isolation provisions for the lines in which these valves are located against the acceptance criteria set forth in the NRC Standard Review Plan regarding the use of a closed system inside containment as an isolation barrier (Section 6.2.4, Paragraph II.6.0). Based on information provided by the licensee, we conclude that these lines meet all the requirements and, therefore, can be considered a closed system. As such, these valves (TV-1BD-100A, B, C and TV-1SS-117A, B, C) should not be subjected to the provisions of Appendix J, Section III.C. These six valves have already been included in the current Inservice Testing Program.

CONCLUSION

Based on the above evaluation, a total of 38 valves are no longer subjected to Type-C test requiremens, and may be leakage tested with water. The licensee should revise his IST program to include all these valves, as well as their test criteria. Table 1 lists all 38 valves.

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

REFERENCES

1. Letter, J. J. Carey (Duquesne Light) to USNRC, dated August 3, 1982.

2. Letter, J. J. Carey (Duquesne Light) to USNRC, dated October 5, 1982.

3. Letter, J. J. Carey (Duquesne Light) to NRC, dated October 25, 1982.

Dated: March 22, 1983

Principal Contributor: R. Palla

Table 1

Valve Number	Service
1CH-31 MOV-1CH-289	Reactor Coolant System Charging
1CH-181, 1CH-182 1CH-183, MOV-1CH-308A MOV-1CH-308B, MOV-1CH-308C	Seal Injection Water to Reactor Coolant Pump
1SI-83, 1SI-84, MOV-1SI-869A, MOV-1SI-869B	High Head Safety Injection to Hot Legs
1SI-451, 1SI-452 1SI-10, 1SI-11, 1SI-12, 1SI-13, 1SI-14, MOV-1SI-890A, MOV-1SI-890B, MOV-1SI-890C	Low Head Safety Injection
1SI-94, 1SI-91, MOV-1SI-867C, MOV-1SI-867D	Boron Injection to Cold Legs
1SI-95, MOV-1SI-836	High Head Safety Injection to Cold Legs
MOV-1SI-860A, MOV-1SI-860B	Low Head Pump Suction from Containment Sump
1CH-170, FCV-1CH-160	Reactor Coolant System Fill
TV-1BD-100A, TV-1BD-100B TV-1BD-100C	Steam Generator Blowdown (Secondary Side)
TV-1SS-117A, TV-1SS-117B, TV-1SS-117C	Steam Generator Blowdown Sample (Secondary Side)

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-334

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 65 to Facility Operating License No. DPR-66 issued to Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees), which revised Technical Specifications for operation of the Beaver Valley Power Station, Unit No. 1 (the facility) located in Beaver County, Pennsylvania. The amendment is effective as of the date of issuance.

The amendment changes the Technical Specifications to waive the need of Type-C testing for 38 valves. These valves may now be water-tested under the licensees' Inservice Testing Program.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated August 3, 1982, supplemented by letters dated October 5, and October 25, 1982, (2) Amendment No. 65 to License No. DPR-66 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the B. F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania 15001. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 22nd day of March, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing