

Docket File  
50-334

October 20, 1980

Docket No. 50-334

CPI

Mr. C. N. Dunn, Vice President  
Operations Division  
Duquesne Light Company  
435 Sixth Avenue  
Pittsburgh, Pennsylvania 15219

Dear Mr. Dunn:

The Commission has issued the enclosed Amendment No. 34 to Facility Operating License No. DPR-66 for the Beaver Valley Nuclear Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letters dated October 17, 1980.

The amendment revises Table 3.7-4 of Radiological Technical Specification 3.7.8.12 by removing nine hydraulic snubbers from this list.

We are taking this opportunity to correct the identification of this Technical Specification, i.e., 3.7.12 rather than 3.7.8.12.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:  
S. A. Varga

Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Enclosures:

1. Amendment No. 34 to DPR-66
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures  
See next page

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October 20, 1980

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

October 20, 1980

Docket No. 50-334

Mr. C. N. Dunn, Vice President  
Operations Division  
Duquesne Light Company  
435 Sixth Avenue  
Pittsburgh, Pennsylvania 15219

Dear Mr. Dunn:

The Commission has issued the enclosed Amendment No. 34 to Facility Operating License No. DPR-66 for the Beaver Valley Nuclear Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letters dated October 17, 1980.

The amendment revises Table 3.7-4 of Radiological Technical Specification 3.7.8.12 by removing nine hydraulic snubbers from this list.

We are taking this opportunity to correct the identification of this Technical Specification, i.e., 3.7.12 rather than 3.7.8.12.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A handwritten signature in dark ink, appearing to read "Steven A. Varga".

Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Enclosures:

1. Amendment No. 34 to DPR-66
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures  
See next page



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 34  
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated October 17, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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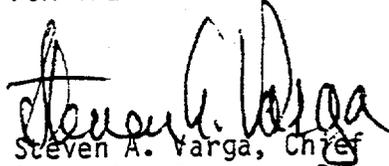
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 34, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Yarga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 20, 1980

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

Remove Pages

3/4 7-26  
3/4 7-27  
3/4 7-32  
3/4 7-32b  
3/4 7-32d

Insert Pages

3/4 7-26  
3/4 7-27  
3/4 7-32  
3/4 7-32b  
3/4 7-32d

## PLANT SYSTEMS

### 3/4.7.11 RESIDUAL HEAT REMOVAL SYSTEM - $T_{avg} < 350^{\circ}\text{F}$

#### LIMITING CONDITION FOR OPERATION

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3.7.11.1 As a minimum, one residual heat removal subsystem shall be OPERABLE.

APPLICABILITY: MODES 4 and 5.

ACTION: With no Residual Heat Removal subsystem OPERABLE, immediately restore at least one RHR subsystem to OPERABLE status or maintain the Reactor Coolant System  $T_{avg}$  less than  $350^{\circ}\text{F}$  by use of alternate heat removal methods.

#### SURVEILLANCE REQUIREMENTS

---

4.7.11.1 The Residual Heat Removal subsystem shall be demonstrated OPERABLE per the applicable Surveillance Requirements of Specification 4.7.10.1.

## PLANT SYSTEMS

### 3/4.7.12 HYDRAULIC SNUBBERS

#### LIMITING CONDITION FOR OPERATION

---

3.7. .12 All hydraulic snubbers listed in Table 3.7-4 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one or more hydraulic snubbers inoperable, restore the inoperable snubber(s) to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

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4.7. .12.1 Each hydraulic snubber with seal material fabricated from ethylene propylene or other materials demonstrated compatible with the operating environment and approved as such by the NRC, shall be determined OPERABLE at least once after not less than 4 months but within 6 months of initial criticality and in accordance with the inspection schedule of Table 4.7-4 thereafter, by a visual inspection of the snubber. Visual inspections of the snubbers shall include, but are not necessarily limited to, inspection of the hydraulic fluid reservoirs, fluid connections, and linkage connections to the piping and anchors. Initiation of the Table 4.7-4 inspection schedule shall be made assuming the unit was previously at the 6 month inspection interval.

4.7. .12.2 Each hydraulic snubber with seal material not fabricated from ethylene propylene or other materials demonstrated compatible with the operating environment shall be determined OPERABLE at least once per 31 days by a visual inspection of the snubber. Visual inspections of the snubbers shall include, but are not necessarily limited to, inspection of the hydraulic fluid reservoirs, fluid connections, and linkage connections to the piping and anchors.

PLANT SYSTEMS

HYDRAULIC SNUBBERS (Continued)

SURVEILLANCE REQUIREMENTS (Continued)

4.7. .12.3 During shutdown, 18 months after initial criticality and at least once per 18 months thereafter, a representative sample of at least 10 snubbers or at least 10% of all snubbers listed in Table 3.7-4, whichever is less, shall be selected and functionally tested to verify correct piston movement, lock up and bleed. Snubbers selected for functional testing shall be selected on a rotating basis except snubbers identified in Table 3.7-4 as either "Especially Difficult to Remove" or in "High Radiation Zones" may be exempted from functional testing provided these snubbers were demonstrated OPERABLE during previous functional tests. Snubbers found inoperable during functional testing shall be restored to OPERABLE status prior to resuming operation. For each snubber found inoperable during these functional tests, an additional minimum of 10% of all snubbers or 10 snubbers, whichever is less, shall also be functionally tested until no more failures are found or all snubbers have been functionally tested.

TABLE 3.7-4

SAFETY RELATED HYDRAULIC SNUBBERS\*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>				<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
RC-HC-1A	RC	RCP	Cub. A	736'6"	I	Yes	Yes
RC-HC-2A	RC	RCP	Cub. A	731'2"	"	"	"
RC-HC-3A	"	"	"	736'6"	"	"	"
RC-HC-4A	"	"	"	736'6"	"	"	"
RC-HC-5A	"	"	"	728'2"	"	"	"
RC-HC-6A	"	"	"	736'6"	"	"	"
RC-HC-7A	"	"	"	728'2"	"	"	"
RC-HC-8A	"	"	"	731'2"	"	"	"
RC-HC-9A	"	"	"	766'10"	"	"	"
RC-HC-10A	"	"	"	766'10"	"	"	"
RC-HC-11A	"	"	"	766'10"	"	"	"
RC-HC-12A	"	"	"	766'10"	"	"	"
RC-HC-1B	RC	RCP	Cub. B	728'2"	"	"	"
RC-HC-2B	"	"	"	728'2"	"	"	"
RC-HC-3B	"	"	"	731'2"	"	"	"
RC-HC-4B	"	"	"	736'6"	"	"	"
RC-HC-5B	"	"	"	736'6"	"	"	"
RC-HC-6B	"	"	"	736'6"	"	"	"
RC-HC-7B	"	"	"	736'6"	"	"	"
RC-HC-8B	"	"	"	731'2"	"	"	"

BEAVER VALLEY - UNIT 1

3/4 7-28

Amendment No. 2

TABLE 3.7-4 (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS\*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
RC-ISS-101	RC RCP Cub. A 739'	I	Yes	Yes
RC-ISS-102	" " " 739'	"	"	"
SI-ISS-102A	SI " " 745'	"	"	"
SI-ISS-102B	" " " 745'	"	"	"
SI-ISS-414	" " " 741'	"	"	"
RC-ISS-103	RC RCP Cub. B 739'	"	"	"
RC-ISS-104	" " " 739'	"	"	"
SI-ISS-418	SI " " 741'	"	"	"
RC-ISS-23	RC Reac. Cnt. Bldg. 749'	A	No	No
RC-ISS-105	RC RCP Cub. C 739'	I	Yes	Yes
RC-ISS-106	RC " " 739'	"	"	"
SI-ISS-114A	SI " " 745'	"	"	"
SI-ISS-114B	" " " 745'	"	"	"
SI-ISS-422	" " " 741'	"	"	"
SI-ISS-423	" " " 739'	"	"	"
RC-ISS-22	RC Pressurizer Cub. 784'	A	Yes	Yes
RC-ISS-41A	" " " 784'	A	"	"
RC-ISS-44A	" " " 784'	A	"	"
RS-ISS-235	RS Reac. Cnt. Bldg. 832'	I	No	"
RS-ISS-210	" " " 817'	"	"	"
RS-ISS-211	" " " 817'	"	"	"
RS-ISS-213	" " " 800'	"	"	"
RS-ISS-214	" " " 800'	"	"	"
RS-ISS-222	" " " 814'	"	"	"
RS-ISS-223	" " " 814'	"	"	"
RS-ISS-224	" " " 813'	"	"	"
RS-ISS-225	" " " 813'	"	"	"
WFPD-ISS-201FW	" " " 780'	A	No	No

SEAWAY VALLEY - UNIT 1

S/4 7-32

Amendment No. 2, 1/8 34

TABLE 3.7-4 (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS\*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
RC-HSS-119	RC Reac. Cnt. Bldg. 734'	A	Yes	No
SI-HSS-409	SI " " 729'	"	No	"
SI-HSS-410	" " " 731'	"	"	"
SI-HSS-411	" " " 731'	"	"	"
RS-HSS-201	RS " " 731'	"	Yes	"
RS-HSS-202	" " " 731'	"	"	"
RS-HSS-237	" " " 731'	"	No	"
RS-HSS-238	" " " 731'	"	"	"
RS-HSS-229	" " " 731'	"	"	"
RS-HSS-236	" " " 731'	"	"	"
RS-HSS-234	" " " 726'	"	"	"
CC-HSS-405A	" " " 707'	I	Yes	Yes
CC-HSS-405B	" " " 707'	"	"	"
CC-HSS-407A	" " " 711'	"	"	"
CC-HSS-407B	" " " 711'	"	"	"
RS-HSS-205	RS " " 702'	A	No	No
RS-HSS-206	" " " 702'	"	"	"
RS-HSS-219	" " " 702'	"	"	"
RS-HSS-220	" " " 702'	"	"	"
RS-HSS-207	" " " 702'	"	"	"
RS-HSS-208	" " " 702'	"	"	"
RS-HSS-209	" " " 710'	"	"	"
RS-HSS-215	" " " 715'	"	"	"
RS-HSS-216	" " " 715'	"	"	"
RH-HSS-105	RH " " 704'	I	Yes	Yes
RH-HSS-107	" " " 704'	"	"	"
RH-HSS-108	" " " 704'	"	"	"
RH-HSS-111	" " " 704'	"	"	"

BEAVER VALLEY - UNIT 1

3/4 7-32b

Amendment No. 18, 34

TABLE 3.7-4 (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS\*

<u>SNUBBER NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE</u>	<u>HIGH RADIATION ZONE</u>	<u>ESPECIALLY DIFFICULT TO REMOVE</u>
WR-HSS-304B	RW Cable Vault 728'	A	No	No
WR-HSS-316	" " " 733'	"	"	"
WR-HSS-306	" " " 733'	"	"	"
WR-HSS-308	" " " 731'	"	"	"
WR-HSS-309	" " " 731'	"	"	"
WR-HSS-300	" " " 724'	"	"	"
SI-HSS-522	SI " " 731'	"	"	"
SI-HSS-523A	" " " 731'	"	"	"
SI-HSS-523B	" " " 731'	"	"	"
SI-HSS-521	" " " 731'	"	"	"
SI-HSS-516A	" " " 731'	"	"	"
SI-HSS-516B	" " " 731'	"	"	"
SI-HSS-520	" " " 731'	"	"	"
SI-HSS-515	" " " 731'	"	"	"
SI-HSS-519	" " " 731'	"	"	"
SI-HSS-514	" " " 731'	"	"	"
SI-HSS-512	" " " 738'	"	"	"
SI-HSS-512A	" " " 734'	"	"	"
SI-HSS-511	" " " 738'	"	"	"
SI-HSS-518	" " " 733'	"	"	"
SI-HSS-517	" " " 738'	"	"	"
QS-HSS-504	QS Safeguards Area 741'	"	"	"
QS-HSS-205A	QS " " 735'	"	"	"
QS-HSS-205B	" " " 735'	"	"	"
QS-HSS-202	" " " 737'	"	"	"

BEAVER VALLEY - UNIT 1

3/4 7-32d

Amendment No. 18, 34



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

Introduction

During the extended refueling outage after termination of Fuel Cycle No. 1, Duquesne Light Company (the licensee) has performed extensive design changes to Beaver Valley Power Station Unit No. 1. Many of these changes were made to satisfy the conditions and requirements of the Order for Modification of License which was issued by the Commission on September 30, 1977 related to the adequacy of the Net Positive Suction Head (NPSH). Other modifications were made to satisfy the requirements of IE Bulletins 79-02, 79-07, and 79-14 related to structural and seismic qualifications of pipes and pipe supports. In the process of achieving the desired changes, the licensee has concluded that several hydraulic snubbers should be removed and replaced with different types of supports. By letter dated October 17, 1980, the licensee requested that these changes in snubbers be approved.

Discussion and Evaluation

The hydraulic snubbers listed in Table 3.7-4 of the licensee's Radiological Technical Specifications represent the snubbers required to be operable to ensure that the integrity of the reactor coolant system and all other safety related systems is maintained during and following a seismic or other event initiating dynamic loads. As the result of recent design changes and re-analyses of seismic loads within the plant, the licensee concluded that four hydraulic snubbers should be replaced by mechanical snubbers and five other hydraulic snubbers should be replaced by rigid supports.

By Amendment No. 28 to License DPR-66, dated August 27, 1980, the staff approved the design changes required to upgrade the NPSH system. The reanalyses of pipe stresses required by IE Bulletins 79-07 and 79-14 are currently under review by the NRC. By letter of August 11, 1980, the licensee submitted its final report on the seismic reanalysis of all safety related piping that was originally performed by an unsatisfactory computer program. Among the problems included in this report are the lines with the replaced hydraulic snubbers.

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The staff has reviewed the licensee's report to the extent that we have determined that the licensee's analyses indicates that overstressed conditions would not exist if the hydraulic snubbers were replaced with the indicated supports. We, therefore, conclude that the changes are acceptable. However, the licensee's analyses of piping systems under I&E Bulletin 79-14 is continuing and further staff review may be performed if warranted.

#### Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 20, 1980

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-334DUQUESNE LIGHT COMPANYOHIO EDISON COMPANYPENNSYLVANIA POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 34 to Facility Operating License No. DPR-66 issued to Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees), which revised Technical Specifications for operation of the Beaver Valley Power Station, Unit No. 1 (the facility) located in Beaver County, Pennsylvania. The amendment is effective as of the date of issuance.

The amendment revises Table 3.7-4 of Radiological Technical Specification 3.7.8.12 by removing nine hydraulic snubbers from this list.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

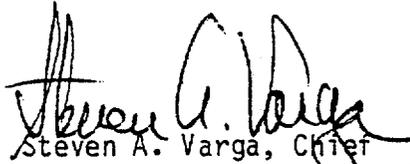
- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated October 17, 1980, (2) Amendment No. 34 to License No. DPR-66 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the B. F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania 15001. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 20th day of October 1980.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing