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November 10, 1982

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Docket No. 50-334

Mr. J. J. Carey, Vice President
Nuclear Division
Duquesne Light Company
435 Sixth Avenue
Pittsburgh, Pennsylvania 15219

Dear Mr. Carey:

The Commission has issued the enclosed Amendment No. 59 to Facility Operating License No. DPR-66 for Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated July 30, 1982.

The amendment changes the Technical Specification on radiation monitoring instrumentation by adding the high range containment area radiation monitors and noble gas effluent monitors. The inclusion of these monitors is recommended in NUREG-0737, Items II.F.1.1, II.F.1.2 and II.F.1.3.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Peter S. Tam, Project Manager
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:

- 1. Amendment No. 59 to DPR-66
- 2. Safety Evaluation
- 3. Notice of Issuance

cc w/enclosures:

See next page

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see suggestions marked
by paper clip.*

OFFICE	ORB 1	ORB 1	ORB 1	AD:OR:DL	OELD		
SURNAME	CParrish	PTam	SVarda	GLainas	JHGuthery		
DATE	10/1/82	10/27/82	10/1/82	10/1/82	10/8/82		

Mr. J. J. Carey
Duquesne Light Company

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Mr. J. J. Carey
Duquesne Light Company

cc: Regional Radiation Representatives
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 59
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated July 30, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 59, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 10, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 3-34	3/4 3-34
-	3/4 3-34a
3/4 3-35	3/4 3-35
3/4 3-36	3/4 3-36
-	3/4 3-36a
B 3/4 3-2	B 3/4 3-2

Revise Appendix B as follows:

Table 2.4-4

Table 2.4-4

TABLE 3.3-6

RADIATION MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>SETPOINT#</u>	<u>MEASUREMENT RANGE</u>	<u>ACTION</u>
1. AREA MONITORS					
a. Fuel Storage Pool Area (RM-207)	1	*	≤15 mR/hr	10 ⁻¹ - 10 ⁴ mR/hr	19
b. Containment					
i. Purge & Exhaust Isolation (RMVS 104 A & B)	1	6	≤1.6 x 10 ³ cpm	10 - 10 ⁶ cpm	22
ii. Area (RM-RM-219 A & B)	2	1, 2, 3, & 4	≤30 R/hr	1 - 10 ⁷ R/hr	36
2. PROCESS MONITORS					
a. Containment					
i. Caseous Activity					
Purge & Exhaust Isolation (RM-215B)	1	6	≤7.3 x 10 ² cpm	10 - 10 ⁶ cpm	22
RCS Leakage Detection (RM 215B)	1	1, 2, 3, & 4	N/A	10 - 10 ⁶ cpm	20
ii. Particulate Activity					
Purge & Exhaust Isolation (RM 215A)	1	6	≤2.5 x 10 ⁵ cpm	10 - 10 ⁶ cpm	22
RCS Leakage Detection (RM 215A)	1	1, 2, 3, & 4	N/A	10 - 10 ⁶ cpm	20
b. Fuel Storage Building Gross Activity (RMVS - 103 A & B)	1	**	≤4.0 x 10 ⁴ cpm	10 - 10 ⁶ cpm	21

* With fuel in the storage pool or building

* With irradiated fuel in the storage pool

Above background

TABLE 3.3-6 (Continued)

RADIATION MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>SETPOINT#</u>	<u>MEASUREMENT RANGE</u>	<u>ACTION</u>
PROCESS MONITORS (Continued)					
c. Noble Gas Effluent Monitors					
i. Supplementary Leak Collection and Release System (RM-VS-110 Ch. 7 & Ch. 9) ***	1	1, 2, 3, & 4	$\leq 3.5 \times 10^2$ cpm	$10^{-2} - 10^5$ uCi/cc*	36
ii. Auxiliary Building Ventilation System (RM-VS-109 Ch. 7 & Ch. 9) ***	1	1, 2, 3, & 4	$\leq 2.75 \times 10^2$ cpm	$10^{-2} - 10^5$ uCi/cc*	36
iii. Process Vent System (RM-GW-109 Ch. 7 & Ch. 9) ***	1	1, 2, 3, & 4	$\leq 1.8 \times 10^4$ cpm	$10^{-2} - 10^5$ uCi/cc**	36
iv. Atmospheric Steam Dump Valve and Code Safety Relief Valve Discharge (RM-MS-100 A, B, C)	1/S.G	1, 2, 3, & 4	$\leq 5.0 \times 10^1$ cpm	$10^{-1} - 10^3$ uCi/cc	36
v. Auxiliary Feedwater Pump Turbine Exhaust (RM-MS-101)	1	1, 2, 3, & 4	$\leq 6.5 \times 10^2$ cpm	$10^{-1} - 10^3$ uCi/cc	36

* Nominal range for Ch. 7 and Ch. 9. Alarm set on Ch. 7

** Nominal range for Ch. 7 and Ch. 9. Alarm set on Ch. 9

*** Other SPING-4 channels not applicable to this specification

Above background

TABLE 3.3-6 (Continued)

TABLE NOTATION

- ACTION 19 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, perform area surveys of the monitored area with portable monitoring instrumentation at least once per 24 hours.
- ACTION 20 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.4.6.1.
- ACTION 21 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the applicable ACTION requirements of Specifications 3.9.12 and 3.9.13.
- ACTION 22 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.9.9.
- ACTION 36 - With the number of OPERABLE channels less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable Channel(s) to OPERABLE status within 72 hours,
or:
- 1) Initiate the preplanned alternate method of monitoring the appropriate parameter(s), and
 - 2) Prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

TABLE 4.3-3

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
1. AREA MONITORS				
a. Fuel Storage Pool Area (RM 207)	S	R	M	*
b. Containment				
i. Purge & Exhaust Isolation (RMVS 104 A & B)	S	R	M	6
ii. Area (RM-RM-219 A & B)	S	R	M	1, 2, 3, & 4
2. PROCESS MONITORS				
a. Containment				
i. Gaseous Activity Purge & Exhaust Isolation (RM 215B) RCS Leakage Detection (RM 215B)	S	R	M	6
	S	R	M	1, 2, 3, & 4
ii. Particulate Activity Purge & Exhaust Isolation (RM 215A) RCS Leakage Detection (RM 215A)	S	R	M	6
	S	R	M	1, 2, 3, & 4
b. Fuel Storage Building Gross Activity (RMVS 103 A & B)	S	R	M	**

* With fuel in the storage pool or building
 ** With irradiated fuel in the storage pool

BEAVER VALLEY - UNIT 1

3/4 3-36

AMENDMENT NO. 59

TABLE 4.3-3 (Continued)

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE IS REQUIRED</u>
PROCESS MONITORS (Continued)				
c. Noble Gas Effluent Monitors				
i. Supplementary Leak Collection and Release System (RM-VS-110 Ch. 7 & Ch. 9)	S	R	M	1, 2, 3, & 4
ii. Auxiliary Building Ventilation System (RM-VS-109 Ch. 7 & Ch. 9)	S	R	M	1, 2, 3, & 4
iii. Process Vent System (RM-GW-109 Ch. 7 & Ch. 9)	S	R	M	1, 2, 3, & 4
iv. Atmospheric Steam Dump Valve and Code Safety Relief Valve Discharge (RM-MS-100 A, B, C)	S	R	M	1, 2, 3, & 4
v. Auxiliary Feedwater Pump Turbine Exhaust (RM-MS-101)	S	R	M	1, 2, 3, & 4

BEAVER VALLEY - UNIT 1

3/4 3-36a

AMENDMENT NO. 59

INSTRUMENTATION

BASES

3/4.3.3 MONITORING INSTRUMENTATION

3/4.3.3.1 RADIATION MONITORING INSTRUMENTATION

The OPERABILITY of the radiation monitoring channels ensures that: 1) the radiation levels are continually measured in the areas served by the individual channels; 2) the alarm or automatic action is initiated when the radiation level trip setpoint is exceeded; and 3) sufficient information is available on selected plant parameters to monitor and assess these variables following an accident. This capability is consistent with the recommendations of NUREG-0737, "Clarification of TMI Action Plan Requirements," October, 1980.

3/4.3.3.2 MOVABLE INCORE DETECTORS

The OPERABILITY of the movable incore detectors with the specified minimum complement of equipment ensures that the measurements obtained from use of this system accurately represent the spatial neutron flux distribution of the reactor core. The OPERABILITY of this system is demonstrated by irradiating each detector used and determining the acceptability of its voltage curve.

For the purpose of measuring $F_0(Z)$ or F_{2H}^N , a full incore flux map is used. Quarter-core flux maps, as defined in WCAP-8648, June 1976, may be used in recalibration of the excore neutron flux detection system, and full incore flux maps or symmetric incore thimbles may be used for monitoring the Quadrant Power Tilt Ratio when one Power Range Channel is inoperable.

3/4.3.3.3 SEISMIC INSTRUMENTATION

The OPERABILITY of the seismic instrumentation ensures that sufficient capability is available to promptly determine the magnitude of a seismic event and evaluate the response of those features important to safety. This capability is required to permit comparison of the measured response to that used in the design basis for the facility and is consistent with the recommendations of Regulatory Guide 1.12, "Instrumentation for Earthquakes."

3/4.3.3.4 METEOROLOGICAL INSTRUMENTATION

The OPERABILITY of the meteorological instrumentation ensures that sufficient meteorological data is available for estimating potential radiation doses to the public as a result of routine or accidental release of radioactive materials to the atmosphere. This capability is required to evaluate the need for initiating protective measures to protect the health and safety of the public and is consistent with the recommendations of Regulatory Guide 1.23, "Onsite Meteorological Programs."

TABLE 2.4-4

PRESSURIZED WATER REACTOR GASEOUS WASTE SYSTEM
LOCATION OF PROCESS AND EFFLUENT MONITORS AND SAMPLERS REQUIRED BY TECHNICAL SPECIFICATIONS

Process Stream or Release Point	Alarm ^b	Auto Control to Isolation Valve or Filters	Continuous Monitor	Grab Sample Station	Measurement				
					Noble Gas	I	Particulate	H-3	Alpha
Waste Gas Storage Tanks ⁽²⁾ RM-GW-101	X	X	X	X	X	X	X	X	X
Condenser Air Ejector ^{a(2)} RM-SV-100	X	X	X		X	X	X	X	X
Vent Header System ^{a(2)}	X		X	X	X	X	X	X	X
Atmospheric Steam Dump Valve and Code Safety Relief Valve Discharge			X		X				
Auxiliary Feedwater Pump Turbine Exhaust			X		X				
<u>Building Ventilation Systems</u>									
Reactor Containment Building (whenever there is flow) ^{a(1)} or (3) RM-VS-105	X	X	X	X	X	X	X	X	X
Auxiliary Building ^{a(3)} or (1) RM-VS-102 A & B	X		X	X	X	X	X	X	X
Lab Hoods and PCA Shop ^{a(3)}	X		X	X	X	X	X	X	X
<u>Supplementary Leak Collection System</u>									
Fuel Building ^{a(1)} RM-VS-103 A & B	X		X	X	X	X	X	X	X
Containment Vacuum Pumps ^{a(2)}	X		X	X	X	X	X	X	X

^aIf any or all of the process streams or building ventilation systems are routed to a single release point, the need for a continuous monitor at the individual discharge point to the main exhaust duct is eliminated. One continuous monitor and measurement capability at the final release point is sufficient.

(1) Elevated release - Top Reactor Containment. RM-VS-107 A & B

(2) Process Vent - Top Cooling Tower. RM-GW-108 A & B

(3) Ventilation Vent - Roof Aux. Bldg. RM - VS-101 A & B

^bAlarms are at downstream locations, not necessarily specific to the stated process steam, but to its final release point [footnote a(1), a(2) and a(3)]



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

Introduction

By letter dated July 30, 1982, Duquesne Light Company (the licensee) proposed to amend its Facility Operating License DPR-66 for Beaver Valley, Unit No. 1, by submitting a revision to the Technical Specifications.

The licensee proposed to modify Table 3.3-6 entitled, "Radiation Monitoring Instrumentation," of Limiting Conditions for Operation (LCO) 3.3.3, and Table 4.3-3 entitled, "Radiation Monitoring Instrumentation Surveillance Requirements," of Surveillance Requirement 4.3.3. He also proposed to modify the Bases for Section 3/4.3.3.1.

Discussion

The licensee proposed that Table 3.3-6 be expanded to include the containment area radiation monitors and the noble gas effluent monitors which monitor the supplemental leak collection and release system, the auxiliary building ventilation system, the process vent system, the atmospheric steam dump valve and code safety relief valve discharge, and the auxiliary feedwater pump turbine exhaust. These monitors were proposed for addition as a part of the implementation of TMI Action Plan Requirements, Attachments 1, 2, and 3 of Item II.F.1 of NUREG-0737. The licensee also proposed to modify Table 4.3-3 so that the surveillance requirements of this Table would be expanded to include the monitors proposed for addition to Table 3.3-6. He also proposed that the Bases for Section 3/4.3.3.1 be revised to reflect the additional capability of the new instrumentation to monitor various plant areas and process systems and for the revised Bases to indicate that this capability was in response to the requirements of NUREG-0737.

Evaluation

We have reviewed the licensee's proposed changes against the guidance provided by Item II.F.1, Attachments 1-3 of NUREG-0737. The range of the noble gas effluent monitors are consistent with the recommendations of Table II.F.1-1. The licensee did not file any request for deviations from the position of II.F.1, Attachment 2,

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or to the clarification associated with this attachment. Therefore, we find that the system meets the guidance of II.F.1, Attachment 2. The licensee should have available for review, by NRC inspectors, the final design details of the implementation of the position and clarification of Attachment 2. It should be noted that the releases from the atmospheric steam dump valves, the code safety relief valves, and the auxiliary feedwater turbine exhausts will not be sampled for radioiodines and particulates due to the high steam content in the release.

We have reviewed the addition of the containment high-range radiation monitors to Tables 3.3-6 and 4.3-3 and have determined that the addition of these monitors is in accordance with Attachment 3 of Item II.F.1 and the Standard Technical Specifications.

We have determined that the licensee needs to add the noble gas effluent monitors associated with the auxiliary feedwater turbine exhaust and the atmospheric steam dump valve and code safety relief valve discharge to Table 2.4-4 of the Environmental Technical Specifications. We have discussed this addition with the licensee and he concurs with this recommendation. With this addition, the proposed changes to Tables 3.3-6 and 4.3-3, and the Bases of Section 3/4.3.3.1 are considered acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 10, 1982

Principal Contributors

J. J. Hayes
R. Serbu

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-334DUQUESNE LIGHT COMPANYOHIO EDISON COMPANYPENNSYLVANIA POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 59 to Facility Operating License No. DPR-66 issued to Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees), which revised Technical Specifications for operation of the Beaver Valley Power Station, Unit No. 1 (the facility) located in Beaver County, Pennsylvania. The amendment is effective as of the date of issuance.

The amendment adds Technical Specifications on radiation monitoring instrumentation, as recommended by NUREG-0737, Items II.F.1.1, II.F.1.2, and II.F.1.3.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated July 30, 1982, (2) Amendment No. 59 to License No. DPR-66 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the B. F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania 15001. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 10 day of November 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing