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Docket No. 50-334

DEC 28 1982

Mr. J. J. Carey, Vice President
 Duquesne Light Company
 Nuclear Division
 Post Office Box 4
 Shippingport, Pennsylvania 15077

Dear Mr. Carey:

The Commission has issued the enclosed Amendment No. 60 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated February 11, 1982. Our review of your Appendix R submittal is being handled separately.

Within our limited scope of the review, we conclude that the requested Technical Specification change is technically sound and responsive to the concerns raised in our Safety Evaluation Report dated May 3, 1979. The requested change is approved. This amendment modifies Table 3.3-10 of your Technical Specifications to reflect the fire detection instruments that have been added to Beaver Valley Unit No. 1.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

Peter S. Tam, Project Manager
 Operating Reactors Branch #1
 Division of Licensing

Enclosures:

1. Amendment No. 60 to DPR-66
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
 See next page

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DATE	12/15/82	12/14/82	12/11/82	12/15/82	12/23/82		

Mr. J. J. Carey
Duquesne Light Company

cc: Mr. W. S. Lacey
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Mr. J. J. Carey
Duquesne Light Company

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated February 11, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

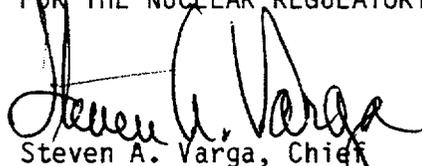
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 60, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 28, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 60 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

Remove Page

3/4 3-48

Insert Pages

3/4 3-48

3/4 3-48a

TABLE 3.3-10

FIRE DETECTION INSTRUMENTS

<u>Instrument Location</u>	<u>Minimum Instruments Operable</u>	
	<u>Smoke</u>	<u>Heat</u>
1. Control Room	4	N/A
2. Cable Spreading Mezzanine	20	4
3. West Cable Vault	3	3
4. East Cable Vault	3	3
5. Computer Room	1	N/A
6. Normal Switchgear Room	8	N/A
7. A/E Emergency Switchgear Room	3	N/A
8. D/F Emergency Switchgear Room	3	N/A
9. Remote Shutdown Panel (Process Instrument Room)	1	N/A
10. Station Battery Rooms (each)	N/A	1
11. Relay Room	1	N/A
12. No. 1 Diesel Generator	2	1
13. No. 2 Diesel Generator	2	1
14. Upper Charcoal Filters	N/A	6
15. Lower Charcoal Filter	N/A	6
16. Control Room Air Conditioning Room	2	N/A
17. Reactor Trip Breaker Room	3	N/A
18. Unit II Control Room		
Zone 7	4	N/A
Zone 8	1	N/A
Zone 9	1	N/A
19. Charging Pump Cubicle	1/cubicle	N/A

TABLE 3.3-10 (continued)

FIRE DETECTION INSTRUMENTS

<u>Instrument Location</u>	<u>Minimum Instruments Operable</u>	
	<u>Smoke</u>	<u>Heat</u>
20. Cable Vault 3 (Elev. 720' on side of Unit 2 Control Room)	4	N/A
21. Intake Structure (A, B and C Cubicle)	6/cubicle	N/A
NOTE: D CUBICLE	N/A	1
22. CCR Pump Area	4	N/A
23. Auxiliary Feedwater Pump Area	4	N/A
24. Cable Penetration Area	2	N/A
25. RHR Pump Area	2	N/A



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 60 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

Introduction

Beaver Valley Power Station Unit 1 is required to have fire detection instruments by its Technical Specifications.

Table 3.3 - 10 of the Unit's Appendix A Technical Specification provides the minimum operable fire detection instruments for the Unit. In May 1979, the NRC conducted a detailed review of the licensee's Fire Protection Program (Reference 1). During this review, the NRC identified six areas of the plant as not having complete fire detection coverage. In light of this identification, the licensee agreed to install additional fire detectors in the identified areas. In addition, the licensee noticed corrosion in the smoke chamber and electronic circuitry associated with the smoke detectors installed in the corrosive environment of the Station Battery Rooms.

The licensee proposed a Technical Specification change (Reference 2) to add detectors in areas determined (by the NRC and the licensee) to have incomplete fire detection coverage and to replace the smoke detectors in battery rooms with heat detectors.

This Safety Evaluation Report (SER) addresses the bases for the proposed Technical Specification change. Since a detailed NRC review of the licensee's Fire Protection Program (Reference 3) is in progress, this SER does not address the fire hazard analyses; detector system design adequacy; fire suppression system design adequacy; and other 10 CFR Part 50, Appendix R, related aspects for the locations stated in the proposed Technical Specification change. A separate SER will be issued to discuss the compliance of the licensee's Fire Protection Program with the requirements of 10 CFR Part 50, Appendix R.

During September 28 - 30, 1982, a NRC Region I reviewer conducted an announced inspection at the Beaver Valley Power Station (Reference 4) to discuss the bases for the proposed Technical Specification change. The inspection included review of licensee's supporting documents, discussions with licensee's cognizant engineers, and independent verification of the licensee's conclusion.

Evaluation

a. Item 10 - Table 3.3 - 10 of Reference 2 Station Battery Rooms

The licensee proposes to replace the existing Honeywell Ionization Smoke detectors with Honeywell Heat Detectors to avoid corrosion problems experienced by the smoke detector installed in the Station Battery Rooms. The licensee's review indicated that the heat detectors were less susceptible to corrosion, as they do not contain smoke chambers and electronic circuitry at the detector location. The manufacturer-recommended detection coverage (2500 sq. ft.) for the heat detector is substantially higher than that for the smoke detector (900 sq. ft.). Since the heat detector belongs to the category of Automatic Fire Detectors defined in NFPA 72E, this replacement meets the intent of the Branch Technical Position CMEB 9.5.1 to have Automatic Fire Detectors in the battery rooms. The replacement can be accomplished by replacing detectors without affecting the associated electrical and mechanical installation. The detector systems for the battery rooms were previously reviewed by NRC in Reference 1 and found acceptable.

b. Items 18 through 25 - Table 3.3 - 10, Reference 2, Additional Areas For Fire Detection Coverage

Locations for items 19, 21, 23, 24, and 25 were identified by the NRC in Reference 1 as areas not having complete fire detection coverage. The remaining areas were added by the licensee. A review of the licensee's supporting documents indicated that the detector systems for these new locations were designed using the criteria for the detector systems previously reviewed and accepted by the Commission in Reference 1.

Within the limited scope of this review and based on the above evaluation, we conclude:

- a. The proposed Technical Specification change meets the intent of NRC Branch Technical Position CMEB 9.5.1.
- b. The licensee's bases for the requested change is technically sound, and
- c. The proposed change would provide better and increased fire detection coverage to plant areas that are important to safety.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made

this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

References

1. NRC Fire Protection Safety Evaluation for Beaver Valley Power Station Unit 1, dated May 3, 1979.
2. Letter from J. J. Carey (Duquesne Light Company) to S. A. Varga (NRC) dated February 11, 1982; subject: Proposed Change Request No. 63 to Operating License.
3. Fire Protection Appendix R Review, Beaver Valley Power Station, dated June 1982.
4. NRC Region I Inspection Report 50-334/82-24 (pages 8 - 10).

Dated: December 28, 1982

Principal Contributor: P.K. Eapen

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-334DUQUESNE LIGHT COMPANYOHIO EDISON COMPANYPENNSYLVANIA POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 60 to Facility Operating License No. DPR-66 issued to Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees), which revised Technical Specifications for operation of the Beaver Valley Power Station, Unit No. 1 (the facility) located in Beaver County, Pennsylvania. The amendment is effective as of the date of issuance.

The amendment modifies Table 3.3-10 to reflect the fire detection instruments that have been added to Beaver Valley Unit No. 1.

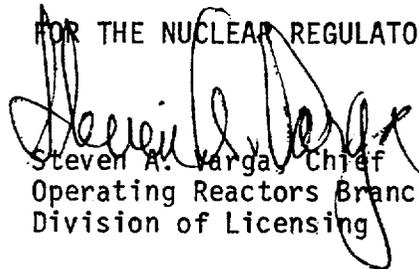
The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated February 11, 1982, (2) Amendment No. 60 to License No. DPR-66 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the B. F. Jones Memorial Library, 663 Aliquippa, Pennsylvania 15001. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 28th day of December, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing