

NOVEMBER 8 1979

Distribution

- Docket File 50-334 I&E (5)
- NRC PDR B. Jones (4)
- Local PDR B. Scharf (10)
- NRR Rdg D. Brinkman
- ORBI Rdg B. Harless
- D. Eisenhut C. Miles
- B. Grimes R. Diggs
- W. Gammill H. Denton
- L. Shao ACRS (16)
- C. Parrish TERA
- D. Wigginton J. Buchanan
- Attorney, OELD
- R. Vollmer

Docket No. 50-334

Mr. C. N. Dunn, Vice President  
 Operations Division  
 Duquesne Light Company  
 435 Sixth Avenue  
 Pittsburgh, Pennsylvania 15219

Dear Mr. Dunn:

The Commission has issued the enclosed Amendment No. 20 to Facility Operating License No. DPR-66 for the Beaver Valley Nuclear Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated September 22, 1978, as supplemented by letter dated February 8, 1979 and portions of letters dated October 27, 1978 and December 14, 1978.

The amendment provides for changes in the nuclear enthalpy hot channel factor, core radial peaking factor, and rod bow penalty. It also deletes the licensing condition 2.C on Rod Bow Penalty on Total Peaking Factor.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original Signature

A. Schwencer, Chief  
 Operating Reactors Branch #1  
 Division of Operating Reactors

Enclosures:

1. Amendment No. 20 to DPR-66
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures  
 See next page

*CP*  
*60*  
*Concern as to form of notice and gridnet only*

*AB 11/3/79*

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\*SEE PREVIOUS YELLOW FOR CONCURRENCE

TAC 10837

OFFICE	DOR:ORBI*	DOR:ORBI*	DOR:ORBI*	DOR:STSG	DOR:ADSORP	OELD
SURNAME	DWigginton	CParrish	ASchwender	DBrinkman	WPGammill	S.SCHINK
DATE	10/16/79	10/17/79	10/21/79	10/23/79	10/29/79	10/31/79

Distribution

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The amendment provides for changes in the nuclear enthalpy hot channel factor, core radial peaking factor, and rod bow penalty.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

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A. Schwencer, Chief  
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OFFICE >	DOR:ORB1	DOR:ORB1	DOR:ORB1	DOR:STSG	DOR:AD:ORP	<del>OR</del> OELD
SURNAME >	DWigginton:jp	CSParrish	ASchwencer	DBrinkman	WPGammill	
DATE >	10/16/79	10/17/79	10/ /79	10/ /79	10/ /79	10/ /79



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

November 3, 1979

Docket No. 50-334

Mr. C. N. Dunn, Vice President  
Operations Division  
Duquesne Light Company  
435 Sixth Avenue  
Pittsburgh, Pennsylvania 15219

Dear Mr. Dunn:

The Commission has issued the enclosed Amendment No. 20 to Facility Operating License No. DPR-66 for the Beaver Valley Nuclear Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated September 22, 1978, as supplemented by letter dated February 8, 1979 and portions of letters dated October 27, 1978 and December 14, 1978.

The amendment provides for changes in the nuclear enthalpy hot channel factor, core radial peaking factor, and rod bow penalty. It also deletes the licensing condition 2.C on Rod Bow Penalty on Total Peaking Factor.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

Enclosures:

1. Amendment No. 20 to DPR-66
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures  
See next page

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Mr. C. N. Dunn  
Duquesne Light Company

- 2 -

November 3, 1979

cc: Gerald Charnoff, Esquire  
Jay E. Silberg, Esquire  
Shaw, Pittman, Potts and Trowbridge  
1800 M Street, N.W.  
Washington, D. C. 20036

Karin Carter, Esquire  
Special Assistant Attorney General.  
Bureau of Administrative Enforcement  
5th Floor, Executive House  
Harrisburg, Pennsylvania 17120

Mr. Roger Tapan  
Stone and Webster Engineering  
Corporation  
P. O. Box 2325  
Boston, Massachusetts 02107

Mr. J. D. Woodward  
R & D Center  
Westinghouse Electric Corporation  
Building 7-303  
Pittsburgh, Pennsylvania 15230

B. F. Jones Memorial Library  
663 Franklin Avenue  
Aliquippa, Pennsylvania 15001

Mr. Jack Carey  
Technical Assistant  
Duquesne Light Company  
P. O. Box 4  
Shippingport, Pennsylvania 15077

Mr. R. E. Martin  
Duquesne Light Company  
435 Sixth Avenue  
Pittsburgh, Pennsylvania 15219

Marvin Fein  
Utility Counsel  
City of Pittsburgh  
313 City-County Building  
Pittsburgh, Pennsylvania 15219

Mr. James A. Werling  
Plant Superintendent  
Beaver Valley Power Station  
P. O. Box 4  
Shippingport, Pennsylvania 15077

Department of Environmental  
Resources  
ATTN: Director, Office of  
Radiological Health  
Post Office Box 2063  
Harrisburg, Pennsylvania 17105

Mr. Thomas J. Czerpah  
Mayor of the Burrough of  
Shippingport  
P. O. Box 26  
Shippingport, Pennsylvania 15077

Ohio Edison Company  
c/o Chief Nuclear QA Engineer  
76 South Main Street  
Akron, Ohio 44308

Pennsylvania Power Company  
Ray E. Semmler, President  
One E. Washington Street  
New Castle, Pennsylvania 16103

Ohio Environmental Protection Agency  
Division of Planning  
Environmental Assessment Section  
P. O. Box 1049  
Columbus, Ohio 43216

Office of the Governor  
State of West Virginia  
Charleston, West Virginia 25305

Mr. Carl Frasure  
Committee of State Officials on  
Suggested State Legislation  
Department of Political Science  
West Virginia University  
Morgantown, West Virginia 26505

Mr. C. N. Dunn  
Duquesne Light Company

- 3 -

November 3, 1979

cc: Mr. Joseph H. Mills, Acting Commissioner  
State of West Virginia Department  
of Labor  
1900 Washington Street  
East Charleston, West Virginia 25305

N. H. Dyer, M.D.  
State Director of Health  
State Department of Health  
State Office Building No. 1  
1800 Washington Street, East  
Charleston, West Virginia 25305

Director, Technical Assessment Division  
Office of Radiation programs (AW-459)  
U. S. Environmental Protection Agency  
Crystal Mall #2  
Arlington, Virginia 20460

U. S. Environmental Protection Agency  
Region III Office  
ATTN: EIS COORDINATOR  
Curtis Building - 6th Floor  
Philadelphia, Pennsylvania 19106

Governor's Office of State Planning  
and Development  
ATTN: Coordinator, Pennsylvania  
State Clearinghouse  
P. O. Box 1323  
Harrisburg, Pennsylvania 17120

Mr. John A. Levin  
Public Utility Commission  
P. O. Box 3265  
Harrisburg, Pennsylvania 17120



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 20  
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated September 22, 1978, as supplemented by letter dated February 8, 1979 and portions of letters dated October 27, 1978 and December 14, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 20, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license is also amended by deletion of the license condition 2.C on "Rod Bow Penalty on Total Peaking Factor."
4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 3, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 20

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages  
3/4 2-6a  
3/4 2-8  
B 3/4 2-4  
B 3/4 2-5

## POWER DISTRIBUTION LIMITS

### SURVEILLANCE REQUIREMENTS (Continued)

- b) At least once per 31 EFPD, whichever occurs first.
2. When the  $F_{xy}^C$  is less than or equal to the  $F_{xy}^{RTP}$  limit for the appropriate measured core plane, additional power distribution maps shall be taken and  $F_{xy}^C$  compared to  $F_{xy}^{RTP}$  and  $F_{xy}^L$  at least once per 31 EFPD.
- e. The  $F_{xy}$  limits for RATED THERMAL POWER within specific core planes shall be:
1.  $F_{xy}^{RTP} \leq 1.71$  for all core planes containing bank "D" control rods, and
  2. For unrodded core planes:
    - $F_{xy} \leq 1.68$  up to 2.4 ft. elevation,
    - $F_{xy} \leq 1.75$  from 2.4 ft. elevation up to 7.8 ft. elevation,
    - $F_{xy} \leq 1.63$  above 7.8 ft. elevation
- f. The  $F_{xy}$  limits of e, above, are not applicable in the following core plane regions as measured in percent of core height from the bottom of the fuel:
1. Lower core region from 0 to 15%, inclusive.
  2. Upper core region from 85 to 100% inclusive.
  3. Grid plane regions at  $17.8 \pm 2\%$ ,  $32.1 \pm 2\%$ ,  $46.4 \pm 2\%$ ,  $60.6 \pm 2\%$  and  $74.9 \pm 2\%$ , inclusive.
  4. Core plane regions within  $\pm 2\%$  of core height ( $\pm 2.88$  inches) about the bank demand position of the bank "D" control rods.
- g. With  $F_{xy}^C$  exceeding  $F_{xy}^L$ , the effects of  $F_{xy}$  on  $F_Q(Z)$  shall be evaluated to determine if  $F_Q(Z)$  is within its limit.

4.2.2.3 When  $F_Q(Z)$  is measured pursuant to Specification 4.10.2.2, an overall measured  $F_Q(Z)$  shall be obtained from a power distribution map and increased by 3% to account for manufacturing tolerances and further increased by 5% to account for measurement uncertainty.

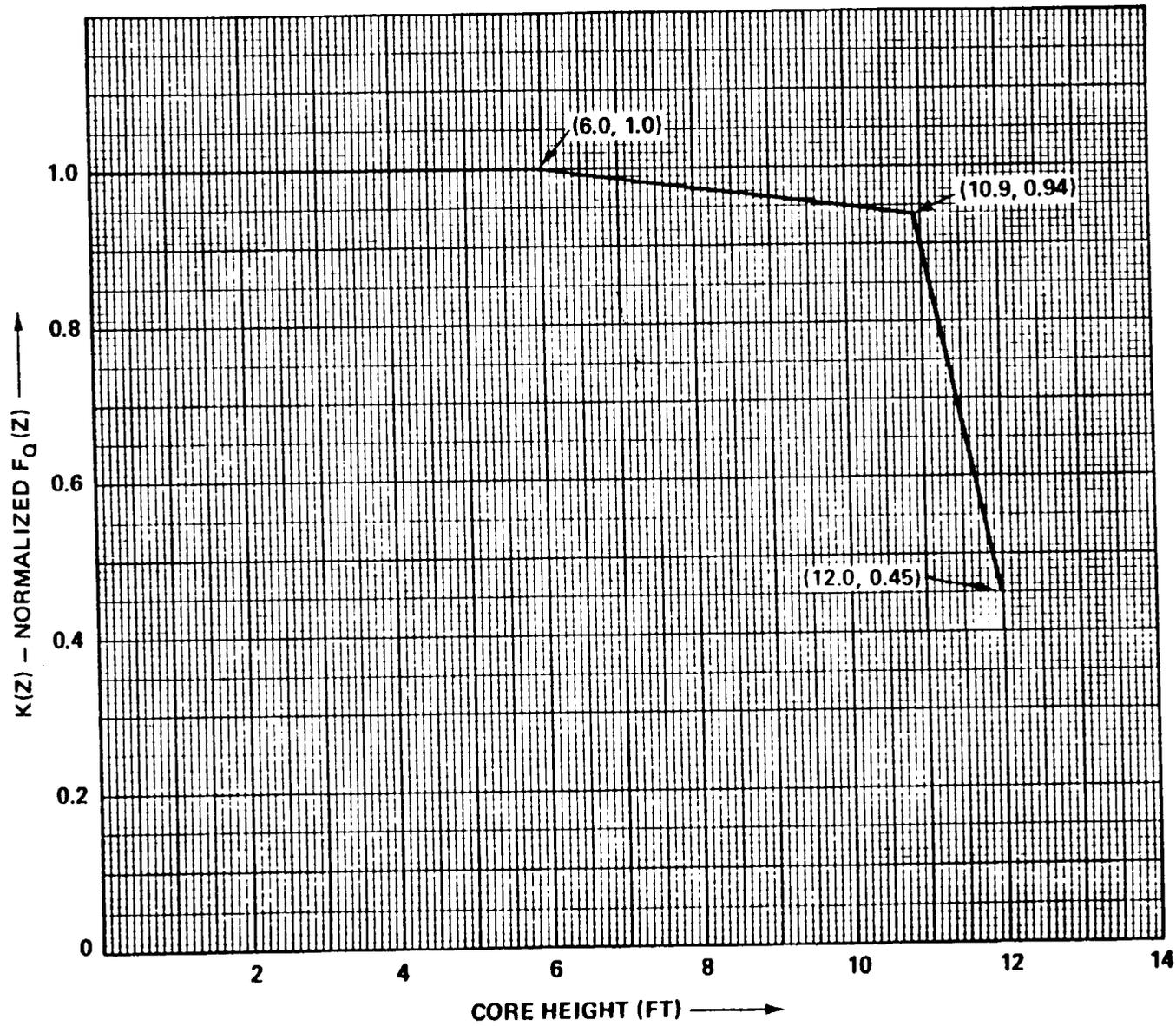


FIGURE 3.2-2

$K(Z)$  - NORMALIZED  $F_0(Z)$   
AS A FUNCTION OF CORE HEIGHT

POWER DISTRIBUTION LIMITS

NUCLEAR ENTHALPY HOT CHANNEL FACTOR -  $F_{\Delta H}^N$

LIMITING CONDITION FOR OPERATION

3.2.3  $F_{\Delta H}^N$  shall be limited by the following relationship:

$$F_{\Delta H}^N \leq 1.5355 [1.0 + 0.2 (1-P)]$$

where  $P = \frac{\text{THERMAL POWER}}{\text{RATED THERMAL POWER}}$

APPLICABILITY: MODE 1

ACTION:

With  $F_{\Delta H}^N$  exceeding its limit:

- a. Reduce THERMAL POWER to less than 50% of RATED THERMAL POWER within 2 hours and reduce the Power Range Neutron Flux-High Trip Setpoints to  $\leq$  55% of RATED THERMAL POWER within the next 4 hours,
- b. Demonstrate thru in-core mapping that  $F_{\Delta H}^N$  is within its limit within 24 hours after exceeding the limit or reduce THERMAL POWER to less than 5% of RATED THERMAL POWER within the next 2 hours, and
- c. Identify and correct the cause of the out of limit condition prior to increasing THERMAL POWER; subsequent POWER OPERATION may proceed provided that  $F_{\Delta H}^N$  is demonstrated through in-core mapping to be within its limit at a nominal 50% of RATED THERMAL POWER prior to exceeding this THERMAL POWER, at a nominal 75% of RATED THERMAL POWER prior to exceeding this THERMAL POWER and within 24 hours after attaining 95% or greater RATED THERMAL POWER.

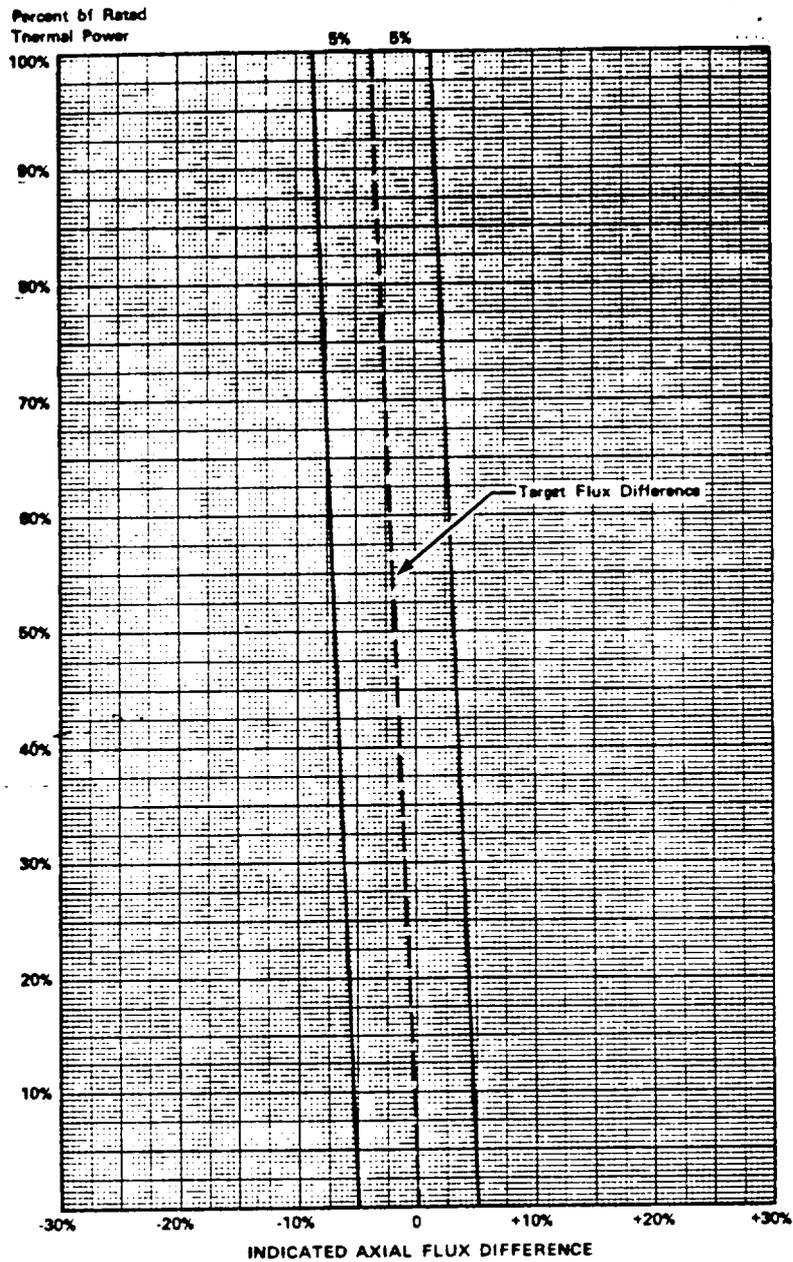


Figure B 3/4 2-1 TYPICAL INDICATED AXIAL FLUX DIFFERENCE VERSUS THERMAL POWER AT BOL

FIGURE B 3/4. 2-1

## POWER DISTRIBUTION LIMITS

### BASES

#### 3/4.2.2 and 3/4.2.3 HEAT FLUX AND NUCLEAR ENTHALPY HOT CHANNEL FACTORS-

$F_Q(Z)$  and  $F_{\Delta H}^N$

The limits on heat flux and nuclear enthalpy hot channel factors ensure that 1) the design limits on peak local power density and minimum DNBR are not exceeded and 2) in the event of a LOCA the peak fuel clad temperature will not exceed the ECCS acceptance criteria limit of 2200°F.

Each of these hot channel factors are measurable but will normally only be determined periodically as specified in Specifications 4.2.2 and 4.2.3. This periodic surveillance is sufficient to insure that the hot channel factor limits are maintained provided:

- a. Control rod in a single group move together with no individual rod insertion differing by more than  $\pm 12$  steps from the group demand position.
- b. Control rod groups are sequenced with overlapping groups as described in Specification 3.1.3.5.
- c. The control rod insertion limits of Specifications 3.1.3.4 and 3.1.3.5 are maintained.
- d. The axial power distribution, expressed in terms of AXIAL FLUX DIFFERENCE is maintained within the limits.

The relaxation in  $F_{\Delta H}^N$  as a function of THERMAL POWER allows changes in the radial power shape for all permissible rod insertion limits.  $F_{\Delta H}^N$  will be maintained within its limits provided conditions a thru d above, are maintained.

When an  $F_Q$  measurement is taken, both experimental error and manufacturing tolerance must be allowed for. 5% is the appropriate allowance for a full core map taken with the incore detector flux mapping system and 3% is the appropriate allowance for manufacturing tolerance.

The specified limit of  $F_{\Delta H}^N$  contains an 8% allowance for uncertainties which means that normal, full  $\Delta H$  power, three loop operation will result in  $F_{\Delta H}^N \leq 1.55/1.08$ .

## POWER DISTRIBUTION LIMITS

### BASES

An evaluation of DNB and test data from experiments of fuel rod bowing in subchannels containing thimble cells has identified that it is appropriate to impose a penalty factor to the accident analyses DNBR results. Accordingly, a thimble cell rod bow penalty as a function of fuel burnup, is applied to the measured values of the enthalpy rise hot channel factor,  $F_{\Delta H}^N$ . This penalty at the end of bundle life, 33000 MWD/MTU is offset by margins available in DNBR analysis and a 1% reduction in  $F_{\Delta H}^N$ .

### 3/4.2.4 QUADRANT POWER TILT RATIO

The quadrant power tilt ratio limit assures that the radial power distribution satisfies the design values used in the power capability analysis. Radial power distribution measurements are made during startup testing and periodically during power operation.

The limit of 1.02 at which corrective action is required provides DNB and linear heat generation rate protection with x-y plane power tilts.

The two hour time allowance for operation with a tilt condition greater than 1.02 but less than 1.09 is provided to allow identification and correction of a dropped or misaligned rod. In the event such action does not correct the tilt, the margin for uncertainty on  $F_0$  is reinstated by reducing the maximum allowed power by 3 percent for each percent of tilt in excess of 1.0.

POWER DISTRIBUTION LIMITS

BASES

3/4.2.5 DNB PARAMETERS

The limits on the DNB related parameters assure that each of the parameters are maintained within the normal steady state envelope of operation assumed in the transient & accident analyses. The limits are consistent with the initial FSAR assumptions and have been analytically demonstrated adequate to maintain a minimum DNBR of 1.30 throughout each analyzed transient.

The 12 hour periodic surveillance of these parameters through instrument readout is sufficient to ensure that the parameters are restored within their limits following load changes and other expected transient operation. The 18 month periodic measurement of the RCS total flow rate is adequate to detect flow degradation and ensure correlation of the flow indication channels with measured flow such that the indicated percent flow will provide sufficient verification of flow rate on a 12 hour basis.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 20 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

Introduction

By letter dated September 22, 1978, Duquesne Light Company (the licensee) requested an amendment to Facility Operating License No. DPR-66. This request was supplemented by letter dated February 8, 1979 and portions of letters dated October 27, 1978 and December 14, 1978. The purpose of the request was to relax the reduction in the enthalpy rise hot channel factor ( $F_{\Delta H}^N$ ) due to fuel rod bowing and to increase the values of radial peaking factor ( $F_{xy}$ ) to elevation dependent values. The  $F_{\Delta H}^N$  factor had been revised previously by Amendment 10 to the Facility Operating License on July 27, 1977.

Evaluation

Relax Existing Reduction in  $F_{\Delta H}^N$  Limit Caused by Fuel Rod Bowing

Westinghouse performed a series of experiments which showed that the DNBR reduction due to fuel rod bowing for a simulated fuel bundle in which the rods were not in contact was less than had been assumed in previous safety analyses. These results are reported in Reference 1. A center rod in a 4 x 4 electrically heated bundle was bowed so that the gap was closed 85% of its nominal spacing. The resulting decrease in DNBR was 10.6%. The staff reviewed these experiments and the resulting calculational model and found these to be acceptable for licensing calculations (Reference 2). To offset the 10.6% reduction in DNBR, Westinghouse has shown that, because of the way they calculated DNBR, there is 9.1% margin available to offset this reduction in DNBR. The residual reduction in DNBR of 1.5% will be accounted for by a reduction of 1% in  $F_{\Delta H}^N$  in the Beaver Valley Technical Specifications. Because the data and calculational model used by the licensee have previously been found acceptable, this change to  $F_{\Delta H}^N$  is acceptable.

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### Core Elevation Dependent Fxy

The licensee has proposed increasing the value of Fxy from 1.55 to an elevation dependent value as follows:

The Fxy limits for RATED THERMAL POWER within specific core planes shall be:

For unrodded core plane:

Fxy < 1.68 up to 2.4 ft. elevation,  
Fxy  $\bar{\leq}$  1.75 from 2.4 ft. elevation up to 7.8 ft. elevation,  
Fxy  $\bar{\leq}$  1.63 above 7.8 ft. elevation.

The licensee has demonstrated by calculation that the use of these values of Fxy results in a Fqx (relative power) less than the current value of  $2.32 \times k(Z)$  (Reference 3). On this basis, and the use by the licensee of acceptable calculational methods, we find this proposed change to Fxy to be acceptable.

### Disposition of License Condition on Rod Bowing

The licensee has requested that a license condition (2C) be removed. This condition requires that, prior to the end of the second fuel cycle, the issue of the effect of fuel rod bowing on  $F_0$  and  $F_{\Delta H}^N$  be resolved. This amendment removes the concern on  $F_{\Delta H}^N$ . The effect on  $F_0$  was resolved by an interim staff SER of April 1976 entitled, "Interim Safety Evaluation Report on Westinghouse Fuel Rod Bowing."\* Therefore the license condition is no longer required and may be removed.

### Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

\*J. R. Reavis, W. J. Leech, F. F. Cadek, S. Arni, and J. M. Hellman, "Fuel Rod Bowing," Westinghouse Licensing Topical Report, WCAP-8691. Interim position transmitted to C. Eicheldinger, Westinghouse by D. Vassallo's letter dated May 11, 1976.

### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

### References

1. Letter from C. Eichelinger, Westinghouse Electric Corporation to Denwood F. Ross, USNRC, NSE-CE-1580, October 24, 1977.
2. Letter from J. F. Stolz, USNRC to T. M. Anderson, Westinghouse Electric Corporation, April 5, 1979.
3. T. Morita, L. R. Scherpereel, K. J. Dzikowski, R. E. Radcliffe, and D. M. Lucoff, "Power Distribution Control and Load Following Procedures," WCAP-8385 (Westinghouse Proprietary), September 1974.

Date: November 3, 1979

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-334DUQUESNE LIGHT COMPANYOHIO EDISON COMPANYPENNSYLVANIA POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 20 to Facility Operating License No. DPR-66 issued to Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees), which revised Technical Specifications for operation of the Beaver Valley Power Station, Unit No. 1 (the facility) located in Beaver County, Pennsylvania. The amendment is effective as of the date of issuance.

The amendment provides for changes in the allowable nuclear enthalpy hot channel factor, and core radial peaking factor and deletes the license condition 2.C on Rod Bow Penalty on Total Peaking Factor.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated September 22, 1978 as supplemented by letter dated February 8, 1979 and portions of letters dated October 27, 1978 and December 14, 1978, (2) Amendment No. 20 to License No. DPR-66 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the B. F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania 15001. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 3rd day of November, 1979.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors