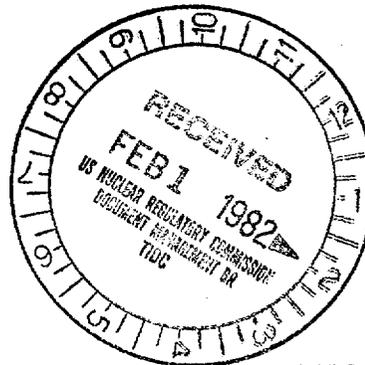


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JAN 27 1982

Docket No. 50-334

Mr. J. J. Carey, Vice President
 Nuclear Division
 Duquesne Light Company
 435 Sixth Avenue
 Pittsburgh, Pennsylvania 15219



Dear Mr. Carey:

The Commission has issued the enclosed Amendment No. 46 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Appendices A and B Technical Specifications in response to your application transmitted by letters dated April 10, June 30, August 13, September 17, and December 22, 1981.

The amendment revises the Appendix A and Appendix B Technical Specifications to reflect the establishment of a Nuclear Division within Duquesne Light Company.

Copies of the Safety Evaluation and Environmental Impact Appraisal and the Notice of Issuance and Negative Declaration are also enclosed.

Sincerely,

ORIGINAL SIGNED

Dennis A. Chaney, Project Manager
 Operating Reactors Branch #1
 Division of Licensing

CP
1

Enclosures:

1. Amendment No. 46 to DPR-66
2. Safety Evaluation and Environmental Impact Appraisal
3. Notice of Issuance and Negative Declaration

cc w/enclosures:
 See next page

*as to item 3
 must + notice
 only*

OFFICE	ORB#1:DL	ORB#1:DL	ORB#1:DL	AD/OR:DL	OELD		
SURNAME	CParrish	DChaney:ds	SVarga	TNYak	Y. Kelly		
DATE	01/15/82	01/15/82	01/15/82	01/15/82	01/20/82		

Mr. J. J. Carey
Duquesne Light Company

cc: Mr. H. P. Williams
Station Superintendent
Duquesne Light Company
Beaver Valley Power Station
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Mr. T. D. Jones, Manager
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Special Assistant Attorney General
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Marvin Fein
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Resident Inspector
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Shippingport, Pennsylvania 15077

Department of Environmental
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Environmental Assessment Section
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Columbus, Ohio 43216

Office of the Governor
State of West Virginia
Charleston, West Virginia 25305

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Thomas and Thomas
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Mr. J. J. Carey
Duquesne Light Company

cc: Regional Radiation Representatives
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Governor's Office of State Planning
and Development
ATTN: Coordinator, Pennsylvania
State Clearinghouse
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Harrisburg, Pennsylvania 17120

Mr. Joseph H. Mills, Acting Commissioner
State of West Virginia Department
of Labor
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East Charleston, West Virginia 25305

N. H. Dyer, M.D.
State Director of Health
State Department of Health
1800 Washington Street, East
Charleston, West Virginia 25305



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 46
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated April 10, 1981 and supplemented June 30, August 13, September 17 and December 22, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

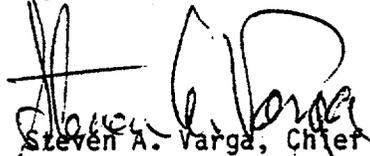
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 46, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 27, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 46 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
6-2	6-2
6-3	6-3
6-5	6-5
6-6	6-6
6-7	6-7
6-8	6-8
6-10	6-10
6-11	6-11
6-12	6-12

Revise Appendix B as follows:

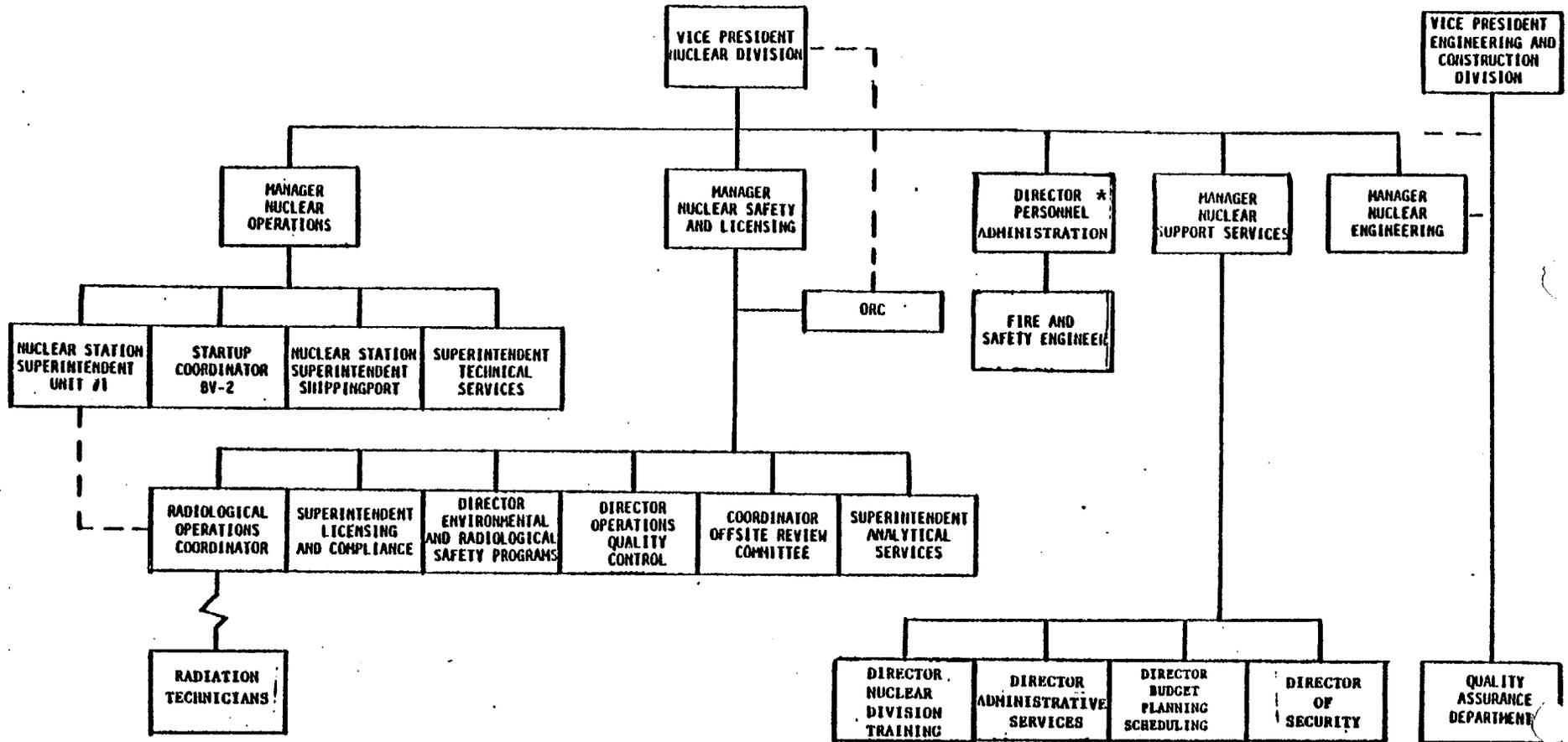
<u>Remove Pages</u>	<u>Insert Pages</u>
5-1	5-1
5-2	5-2
5-3	5-3
5-4	5-4

**DUQUESNE LIGHT COMPANY
Nuclear Division**

BEAVER VALLEY - UNIT 1

6-2

Amendment No. 46



**Figure 6.2-1
Offsite Organization (Partial)**

* Indicates Fire Protection Responsibility

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility and Radiation Protection staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Radiological Operations Coordinator who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design and response analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Director Nuclear Division Training and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A Training program for the Emergency Squad shall be maintained under the direction of the Director Nuclear Division Training and shall meet or exceed the requirements of Section 27 of the NEPA Code-1976.

6.5 REVIEW AND AUDIT

6.5.1 ONSITE SAFETY COMMITTEE (OSC)

FUNCTION

6.5.1.1 The OSC shall function to advise the Station Superintendent on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The OSC shall be composed of the:

Chairman:	Chief Engineer
Member:	Senior Licensed Operator
Member:	Radiation Control Foreman
Member:	Maintenance Engineer
Member:	Senior Engineer - Station Engineering
Member:	Senior Testing or Study Projects Coordinator
Member:	Shift Technical Advisor
Member:	Chemist

NOTE: The chairman of the OSC shall appoint an individual from each of the above listed job categories to serve as a member of the OSC for a period of at least 6 months.

NOTE: OSC members shall meet or exceed the minimum qualifications of ANSI N18.1-1971 Section 4.4 for comparable positions. The SRO shall meet the qualifications of Section 4.2.2 and the Maintenance Engineer will Meet the qualifications of Section 4.2.3.

ADMINISTRATIVE CONTROLS

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the OSC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in OSC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The OSC shall meet at least once per calendar month and as convened by the OSC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the OSC shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The OSC shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Plant Superintendent to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Manager of Nuclear Operations and to the Chairman of the Offsite Review Committee.
- f. Review of those REPORTABLE OCCURRENCES requiring 24 hour notification to the Commission.
- g. Review of facility operations to detect potential safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Offsite Review Committee.

ADMINISTRATIVE CONTROLS

- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Offsite Review Committee.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Offsite Review Committee.

AUTHORITY

6.5.1.7 The OSC Shall:

- a. Recommend to the Plant Superintendent written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Manager of Nuclear Operations and the Offsite Review Committee of disagreement between the OSC and the Plant Superintendent; however, the Plant Superintendent shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The OSC shall maintain written minutes of each meeting and copies shall be provided to the Manager of Nuclear Operations and Chairman of the Offsite Review Committee.

6.5.2 OFFSITE REVIEW COMMITTEE (ORC)

FUNCTION

6.5.2.1 The ORC shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering

ADMINISTRATIVE CONTROLS

- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

COMPOSITION

6.5.2.2 The ORC shall be composed of the:

Chairman:	Manager, Nuclear Safety and Licensing
Vice Chairman:	Manager of Nuclear Engineering, Nuclear Division
Member:	Nuclear Engineer, Mechanical Engineering Department
Member:	Manager of Nuclear Operations
Member:	Instrument Engineer, Power Stations Department
Member:	Supervisor, Central Chemical Laboratory Power Stations Department
Member:	Mechanical Engineer, Mechanical Engineering Department
Member:	Quality Assurance Manager, Quality Assurance Department
Member:	Coordinator, Safety Review Committee, Nuclear Division
Member:	Director, Environmental and Radiological Safety Programs
Member:	Beaver Valley Power Station Project Manager

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the ORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in ORC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the ORC Chairman to provide expert advice to the ORC.

AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the ORC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.
- d. The performance of activities required by the Quality Assurance Program to meet the criteria of Appendix "B," 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 24 months.
- f. The Facility Security Plan and implementing procedures at least once per 24 months.
- g. Any other area of facility operation considered appropriate by the ORC or the Vice President, Nuclear.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outsider fire consultant at least once per 36 months,

AUTHORITY

6.5.2.9 The ORC shall report to the Manager of Nuclear Safety and Licensing and advise the Vice President, Nuclear Division on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of ORC activities shall be prepared, approved and distributed as indicated on the following pages:

ADMINISTRATIVE CONTROLS

- a. Minutes of each ORC meeting shall be prepared, approved and forwarded to the Vice President, Nuclear Division within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President, Nuclear Division within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President, Nuclear Division and to the management positions responsible for the areas audited within 30 days after completion of the audit.
- d. The chairman of the ORC shall review all recommendations of the ORC and shall forward such recommendations to the Vice President, Nuclear Division.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the OSC and submitted to the ORC and the Manager of Nuclear Operations.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least HOT STANDBY within 1 hour.
- b. The Safety Limit violation shall be reported to the Commission, the Manager of Nuclear Operations and to the ORC within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the OSC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the ORC and the Manager of Nuclear Operations within 10 days of the violation.

ADMINISTRATIVE CONTROLS

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the OSC and approved by the Plant Superintendent, predesignated alternate or a predesignated Department Manager to whom the Plant Superintendent has assigned in writing the responsibility for review and approval of specific subjects considered by the committee, as applicable.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the OSC and approved by the Plant Superintendent within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Director of the Regional Office of Inspection and Enforcement unless otherwise noted.

5.0 ADMINISTRATIVE CONTROLS

Objective

To define the organization, assign responsibilities, describe the environmental surveillance procedures, provide for a review and audit function and prescribe the reporting requirements in order to insure continuing protection of the environment and implement the Environmental Technical Specifications.

5.1 ORGANIZATION

Figure 5.1-1 is a partial organization chart of the Nuclear Division which includes groups and individuals responsible for environmental protection, environmental monitoring and the implementation of the Environmental Technical Specifications following the issuance of an operating license for Beaver Valley Unit No. 1.

5.2 RESPONSIBILITY

All activities within the Nuclear Division are the responsibility of the Vice President, Nuclear. He delegates the responsibility for the radiological and ecological environmental monitoring programs to the Manager, Nuclear Safety and Licensing. He is advised by the Off-Site Review Committee in matters relating to the environmental impact of station operation which involve proposed changes to the Environmental Technical Specifications, proposed changes to written procedures, proposed changes or modifications to station systems or equipment, evaluations of investigations, and when appropriate, recommendations to prevent recurrence of such violations.

The Station Superintendent is the individual charged with the responsibility for assuring that all releases to the environment shall be limited in accordance with the Limiting Conditions for Operations as described in the Environmental Technical Specifications. Controlled releases are made utilizing written procedures under the direction of the Shift Supervisor at the request of the Reactor Control Chemist who specifies the release rate, the station operating conditions, and the monitoring requirements which must be maintained during the release to comply with the technical specifications. These activities are reviewed by the Results Coordinator and the Station Operating Supervisor and audited periodically under the direction of the Manager of Nuclear Safety and Licensing.

The Station Superintendent is also responsible for the issuance of the 24-hour reporting requirement to the Director of Regional Regulatory Operations Office, when required.

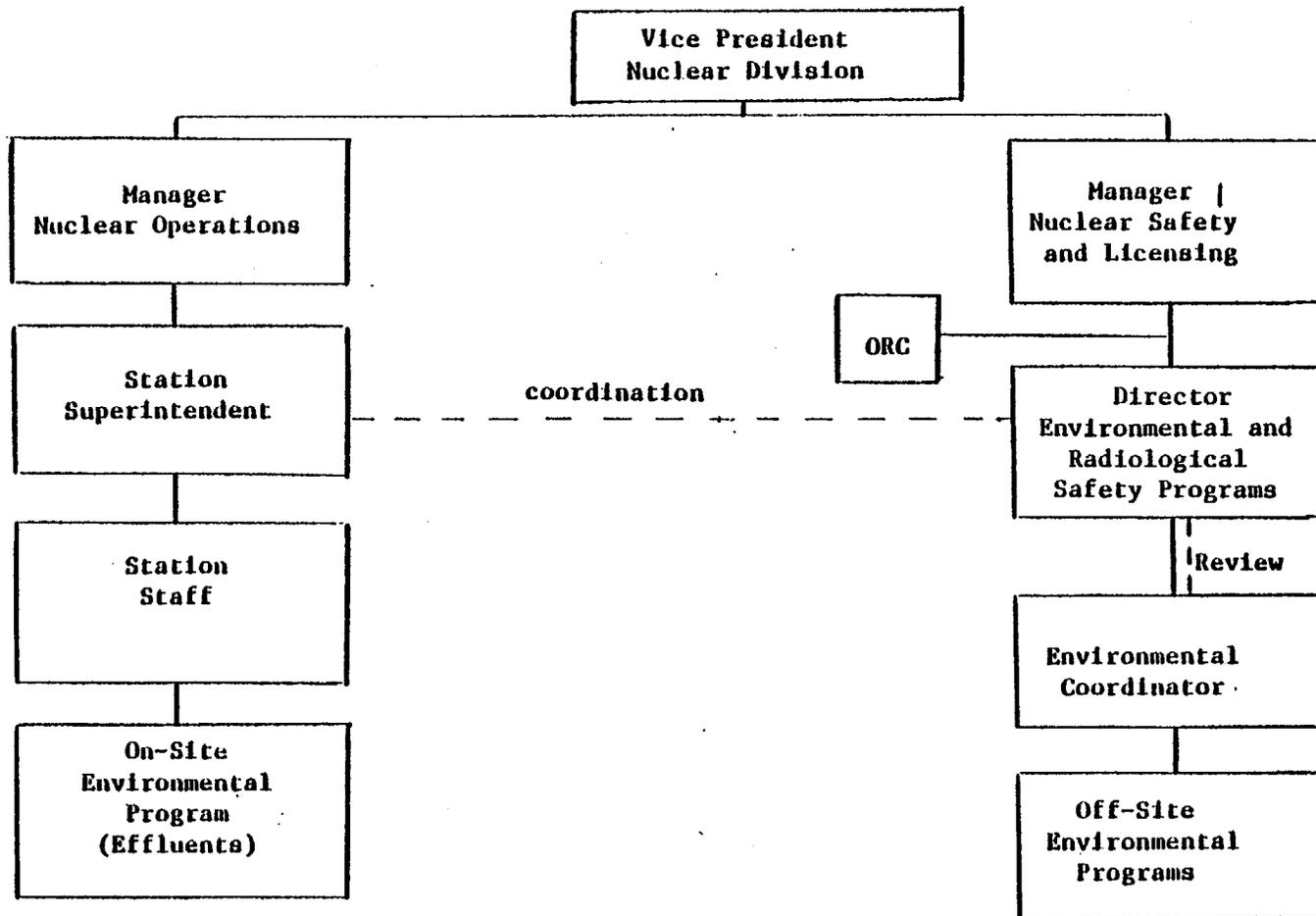


Figure 5.1-1
Duquesne Light Company
Nuclear Division
Partial Organization Chart
Relative to Environmental Matters

5.3 REVIEW AND AUDIT

The Director, Environmental and Radiological Safety Programs is responsible to the Off-Site Review Committee for reviewing the results of the Environmental Surveillance Program to determine that the sampling program is being implemented as described to verify that the environment is adequately protected under the existing station operating procedures. He is also responsible for requesting that analyses be repeated or that new samples be obtained from particular locations in the case of anomalous results.

The Director, Environmental and Radiological Safety Programs is responsible to the Off-Site Review Committee for scheduling audits utilizing various company personnel to verify that station operations including releases to the environment are conducted in such a manner that continuing protection to the environment is provided. The Director, Environmental and Radiological Safety Programs shall also have the responsibility for reviewing off-site environmental program sampling, analyses and calibration procedures as summarized in Table 3.1-3 and 3.2-1 and described in Sections 3.0 and 5.5.1 of these specifications. The Beaver Valley Power Station Superintendent is responsible for approving on-site procedures relative to the environment as described in Tables 2.4-2 and 2.4-4 and described in Section 2.0 and Section 5.5.1 of these specifications.

The Off-Site Review Committee functions as they relate to the Environmental Technical Specifications are as follows:

- a. Review the reports of the Director, Environmental and Radiological Safety Programs on the results of the environmental monitoring program prior to their submittal to the NRC in each Annual Environmental Operating Report. See Section 5.6.1.
- b. Review and make recommendations on proposed changes to the Environmental Technical Specifications and the evaluated impact of the changes.
- c. Review proposed changes or modifications to plant systems or equipment and the evaluated impact which would require a change in the procedures described in (d) below.
- d. Review reported instances of violations of Environmental Technical Specifications and abnormal environmental occurrences. Where investigation indicates, evaluate and formulate recommendations to prevent recurrences.
- e. Schedule audits of station operation with regard to the control of releases to the environment.
- f. Review and compare the Safety Technical Specifications and the Environmental Technical Specifications to avoid conflicts and maintain consistency.

5.4 ACTION TO BE TAKEN IF A LIMITING CONDITION FOR OPERATION IS EXCEEDED

5.4.1 Remedial Action

Immediate remedial actions as permitted by these technical specifications will be implemented until such time as the limiting condition for operation is met.

5.4.2 Investigation

The Manager, Nuclear Safety and Licensing shall direct that an investigation be conducted to determine the cause of the occurrence and recommend both temporary and permanent corrective action to prevent a recurrence. The results of this investigation shall be reviewed and approved by the Off-Site Review Committee and the Manager, Nuclear Operations.

5.4.3 Report

The occurrence shall be reported to the NRC as specified in Section 5.6.2.

5.5 PROCEDURES

5.5.1 Written Procedures

Detailed written procedures, including check lists and special instructions when required, will be prepared and implemented for all activities involving releases to the environment, described in Sections 2.0 and 3.0, the sampling and monitoring of these releases and the conduct of the Environmental Surveillance Program. These procedures will include sampling, instrument calibration, analysis, and actions to be taken when limits are approached or exceeded. Testing and calibration frequencies of effluent alarms and monitoring instruments will be included. These frequencies will be determined from experience with similar instruments in similar environments and from manufacturers' recommendations.

5.5.2 Standard Operating Procedures

Plant standard operating procedures will include, when required, sufficient precaution, limitations, and setpoints to ensure that all plant systems and components are operated in compliance with the limiting conditions for operations established in Section 2.0 of these technical specifications.

5.5.3 Reviews

All procedures described above, and all changes thereto, will be reviewed prior to implementation and periodically thereafter as described in Section 5.3. Temporary changes to procedures which do not change the intent of the original procedure may be made without the need for review by NRC. Changes to procedures for programs in Section 3 will be reviewed by the NRC. Such changes will be documented.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION AND ENVIRONMENTAL IMPACT APPRAISAL

BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 46 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUENSE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

Introduction

Duquesne Light Company, by letters to the NRC dated April 10, 1981, as supplemented June 30, August 13, September 17 and December 22, 1981, requested changes to the Appendix A and Appendix B Technical Specifications for Beaver Valley Power Station, Unit No. 1. These changes reflect the establishment of a Nuclear Division within Duquesne Light Company and include changes in the Onsite Safety Committee (OSC), the Offsite Review Committee (ORC) and the review and approval procedures for plant management.

Discussion

The licensee has proposed to modify its corporate organizational structure to consolidate its major nuclear power plant operation and support activities within a single new corporate division, the Nuclear Division, which is headed by a Vice President whose sole responsibility is for nuclear power related activities. It has also proposed to modify the Beaver Valley Unit 1 plant organizational structure.

The proposed offsite organization reporting to the Vice President - Nuclear Division includes a Manager - Nuclear Operations, Manager - Nuclear Safety and Licensing, Manager - Nuclear Support Services, and Manager - Nuclear Engineering.

The Beaver Valley Unit 1 Station Superintendent reports to the Manager - Nuclear Operations. A new position of Superintendent - Technical Services also reports directly to the Manager - Nuclear Operations. A number of the technical support functions that are currently assigned to the Station Superintendent are assigned to the Superintendent - Technical Services in the new organization. These include the shift technical advisors, refueling, chemistry and station engineering. The current position of Chief Nuclear Engineer has been eliminated from the proposed station organization and the Supervisor - Nuclear Station Operations reports directly to the Station Superintendent. The Instrument and Control function, which is under the Maintenance Supervisor in the current organization, has been set up under a separate Supervisor - Instrument and Control who reports directly to the Station Superintendent. The Supervisor - Maintenance retains the mechanical and electrical maintenance function but reports directly to the Station Superintendent rather than to the Chief Nuclear Engineer.

Other support functions that are under the Station Superintendent in the current organization, including radiation protection, fire protection, nuclear engineering, training, security and office and records are reorganized under the new offsite organization Managers of Nuclear Safety and Licensing, Nuclear Support Services and Nuclear Engineering and the Director, Personnel Administration.

Evaluation

Our evaluation of the proposed organizational changes is based on review of the April 10, June 30, August 13, September 17 and December 22, 1981 submittals from the licensee and conversations with the licensee concerning the proposed changes. It also takes account of a May 7, 1981 NRC Office of Inspection and Enforcement - Region I letter to the licensee concerning an April 3, 1981 meeting with the licensee concerning the proposed organizational changes and an April 2, 1981 letter from the licensee to the NRR Office of Inspection and Enforcement - Region I on the same subject.

On the basis of our evaluation as discussed above, we have concluded that the organizational changes should enhance the ability of the licensee to manage and safely operate the Beaver Valley Power Station Unit 1 and are acceptable.

The licensee has proposed to change the membership of the Onsite Safety Committee (OSC). It would replace the management and supervisory personnel currently designated as members with non-supervisory personnel. It would further designate membership titles that do not identify specific individuals but instead identify a job title or type of license held. Examples of these proposed changes are designation of a Senior Licensed Operator rather than the Operations Supervisor and a Shift Technical Advisor rather than the Technical Advisory Engineer.

Since there are a number of individuals with the titles Senior Licensed Operator and Shift Technical Advisor, these titles alone do not assure that the individuals serving as committee members will conform with the NRC guidance concerning minimum qualifications for committee members. Section 3.4 of the NRC's Standard Review Plan specifies that members of the Onsite Review Committees should meet the qualifications required by Section 4.4 of ANSI N18.1-1971 for professional and technical personnel. We discussed this with the licensee and it added a membership qualifications statement in Section 6.5.1.2 of its September 17, 1981 submittal requiring that OSC members will meet the qualifications specified for professional technical personnel by Section 4.4 of ANSI N18.1-1971.

Since there can be a number of personnel with the titles identified by the proposed membership list, we were concerned that a different individual might attend each meeting and that there would be no continuity in the work performed by the Committee. At our request, the licensee added a statement in Section 6.5.1.2 of its June 30, 1981 submittal that individual members be appointed to serve at least six months. We believe that this will provide adequate continuity to the activities of the OSC.

The licensee's proposed new membership list eliminates several plant disciplines that are included in the current membership list: training, emergency planning, fire protection and security. The proposed list also adds one new discipline: chemistry. As a result of this, the Committee has been decreased by two members. However, we find that the technical disciplines and the number of members proposed is consistent with that currently accepted by the staff for other nuclear stations.

On the basis of our review as discussed above, we conclude that with the changes we have noted above concerning the technical qualifications and the length of appointment terms of members, the changes proposed to the Technical Specifications concerning the OSC are acceptable.

In its submittals to the NRC of March 13, April 10 and June 30, 1981, the licensee has proposed changes to Section 6.5.2.2 - Composition, Section 6.5.2.8 - Audit, Section 6.5.2.9 - Authority and Section 6.5.2.10 - Records. These changes align the designated membership of the Offsite Review Committee (ORC) with the structure and the job titles of the proposed reorganization. The changes also increase the number of designated committee members by two. The proposed new chairman of the ORC is the Manager - Nuclear Safety and Licensing. This position is one step lower in the licensee's management structure than the current chairman who is the General Superintendent of Power Plants. However, the proposed new chairman reports directly to the new Vice President - Nuclear Division, minutes of all ORC meetings are sent to the new Vice President within 14 days following each meeting, and the ORC chairman is required to forward all Committee recommendations to the new Vice President.

We conclude that the proposed changes to Sections 6.5.2.2, 6.5.2.8, 6.5.2.9 and 6.5.2.10 as discussed above will not diminish the effectiveness of the ORC and are acceptable.

Section 6.8.2 of the current Beaver Valley Technical Specification requires that procedures and administrative policies and changes thereto be reviewed by the Onsite Review Committee and approved by the Plant Superintendent prior to implementation.

In its September 17, 1981 submittal, the licensee proposed that Section 6.8.2 be modified to allow procedures and administrative policies and changes thereto to be implemented following review by the Onsite Review Committee and approval by either the Plant Superintendent or the responsible Department Manager. The phrase "or responsible Department Manager" does not adequately describe who shall approve changes.

We believe that the Plant Superintendent should be responsible to assure that appropriate multidisciplinary reviews are performed by the Onsite Review Committee and that the effect on plant safety of all proposed procedures and administrative policies or changes thereto has been appropriately considered. We believe that the Plant Superintendent should be responsible to assure that appropriate review is given to these matters by the affected plant and corporate functional organizations.

We also believe that the Plant Superintendent should be responsible for review and approval of the reports and recommendations developed by the reviewers and for forwarding them, as appropriate, to the independent review group. We recognize that in order to fulfill this responsibility, the Plant General Manager may delegate some of these activities to other specific appropriate plant staff managers. This delegation must be in writing and specific to the particular review activity being performed.

In its submittal to the NRC of December 22, 1981, the licensee changed section 6.8.2 to require that the Plant Superintendent predesignate in writing the Department Manager or other alternate given the responsibility for the review and approval of specific subjects considered by the OSC. We conclude that the proposed revision to Section 6.8.2 is acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: January 27, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-334DUQUESNE LIGHT COMPANYOHIO EDISON COMPANYPENNSYLVANIA POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE
AND NEGATIVE DECLARATION

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 46 to Facility Operating License No. DPR-66 issued to Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees), which revised Appendices A and B Technical Specifications for operation of the Beaver Valley Power Station, Unit No. 1 (the facility) located in Beaver County, Pennsylvania. The amendment is effective as of the date of issuance.

The amendment revises the Appendix A and Appendix B Technical Specifications to reflect the establishment of a Nuclear Division within Duquesne Light Company.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

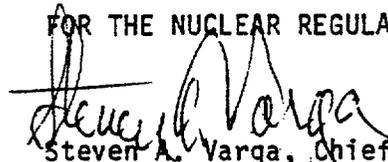
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The Commission has prepared an environmental impact appraisal for the revised Technical Specifications and has concluded that an environmental impact statement for this particular action is not warranted because there will be no environmental impact attributable to the action other than that which has already been predicted and described in the Commission's Final Environmental Statement for the facility dated July 1973.

For further details with respect to this action, see (1) the application for amendment dated April 10, 1981, and supplemented June 30, August 13, September 17 and December 22, 1981, (2) Amendment No. 46 to License No. DPR-66 and (3) the Commission's related Safety Evaluation and Environmental Impact Appraisal. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the B. F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania 15001. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 27th day of January, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing