

MAR 16 1982

Docket No. 50-334

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Mr. J. J. Carey, Vice President  
 Nuclear Division  
 Duquesne Light Company  
 435 Sixth Avenue  
 Pittsburgh, Pennsylvania 15219

Dear Mr. Carey:

The Commission has issued the enclosed Amendment No. 48 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letters dated November 9, 1981.

The amendment revises reactor vessel nozzle to safe end weld inspection requirements and updates the ASME code reference for steam generator tube inspections.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

Dennis A. Chaney, Project Manager  
 Operating Reactors Branch #1  
 Division of Licensing

Enclosures:

1. Amendment No. 48 to DPR-66
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:  
 See next page



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 PDR ADOCK 05000334  
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*cc to person of such  
 + FRB only + submit  
 1 to principal changes*

*DC*

OFFICE	DL:ORB#1 <i>CP</i>	DL:ORB#1	DL:ORB#1	DL:OR	OELD		
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DATE	3/3/82	3/13/82	2/24/82	2/16/82	3/10/82		

Mr. J. J. Carey  
Duquesne Light Company

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Mr. J. J. Carey  
Duquesne Light Company

cc: Regional Radiation Representatives  
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Dr. Clark Hansbarger  
Director  
Health Department  
Building Three, Room 206  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 48  
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated November 9, 1981 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

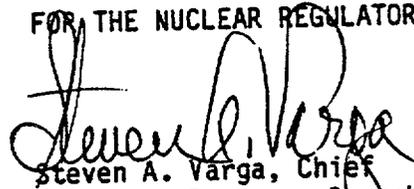
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 48, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 16, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 48 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

Remove Pages

3/4 4-8

3/4 4-29

Insert Pages

3/4 4-8

3/4 4-29

## REACTOR COOLANT SYSTEM

### STEAM GENERATORS

#### LIMITING CONDITION FOR OPERATION

3.4.5 Each steam generator shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one or more steam generators inoperable, restore the inoperable generator(s) to OPERABLE status prior to increasing  $T_{avg}$  above 200°F.

#### SURVEILLANCE REQUIREMENTS

4.4.5.1 Steam Generator Sample Selection and Inspection - Each steam generator shall be determined OPERABLE during shutdown by selecting and inspecting at least the minimum number of steam generators specified in Table 4.4-1.

4.4.5.2 Steam Generator Tube Sample Selection and Inspection - The steam generator tube minimum sample size, inspection result classification, and the corresponding action required shall be as specified in Table 4.4-2. The inservice inspection of steam generator tubes shall be performed at the frequencies specified in Specification 4.4.5.3 and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.4.5.4. Steam generator tubes shall be examined in accordance with the method prescribed in Article 8 - "Eddy Current Examination of Tubular Products," as contained in ASME Boiler and Pressure Vessel Code, Section V - "Non-destructive Examination," and referenced in ASME Boiler and Pressure Vessel Code - Appendix IV of the 1980 Edition through Winter 1980 Addenda of Section XI - "Inservice Inspection of Nuclear Power Plant Components." The tubes selected for each inservice inspection shall include at least 3% of the total number of tubes in all steam generators; the tubes selected for these inspections shall be selected on a random basis except:

- a. Where experience in similar plants with similar water chemistry indicates critical areas to be inspected, then at least 50% of the tubes inspected shall be from these critical areas.
- b. The first inservice inspection (subsequent to the preservice inspection) of each steam generator shall include:
  1. All nonplugged tubes that previously had detectable wall penetrations (>20%), and
  2. Tubes in those areas where experience has indicated potential problems.

## REACTOR COOLANT SYSTEM

### SURVEILLANCE REQUIREMENTS (Continued)

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- a. At the first refueling outage, dye penetrant inspections shall be performed on all six reactor vessel nozzles and volumetric inspections shall be performed on at least two nozzles in the same manner as that conducted in the baseline inspection.
- b. At the second refueling outage, a dye penetrant examination shall be performed on all six nozzle to safe end welds and an ultrasonic examination shall be performed on all six nozzle to safe end welds from the outside diameter.

At the third refueling outage, a dye penetrant examination shall be performed on all six nozzle to safe end welds and an ultrasonic examination shall be performed on all six nozzle to safe end welds from the outside diameter. Ultrasonic examination shall be performed on the three outlet nozzle to safe end welds from the inside diameter.

- c. Defects found during these inspections shall be evaluated to determine their significance and reported to the Commission pursuant to Technical Specification 6.9.1.8.c.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 48 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

INTRODUCTION

Duquesne Light Company, by letter dated November 9, 1981, requested changes to the Appendix A Technical Specifications for Beaver Valley Power Station, Unit No. 1. The changes modify the inspection requirements for the reactor vessel nozzle to safe end welds and updates the ASME code reference for steam generator tube inspections.

DISCUSSION AND EVALUATION

The licensee has requested to change Technical Specification 4.4.10b which reads:

- b. At the second and third refueling outages dye penetrant inspections shall be performed on all six nozzles and volumetric inspections shall be performed on at least two nozzles in the same manner as the baseline inspections so that all six nozzles shall be volumetrically inspected during the first three refueling outages.

to the following:

- b. At the second refueling outage, a dye penetrant examination shall be performed on all six nozzle to safe end welds and an ultrasonic examination shall be performed on all six nozzle-to-safe end welds from the outside diameter.

At the third refueling outage, a dye penetrant examination shall be performed on all six nozzle to safe end welds and an ultrasonic examination shall be performed on all six nozzle-to-safe end welds from the outside diameter. Ultrasonic examination shall be performed on the three outlet nozzle-to-safe end welds from the inside diameter.

### Licensee Basis for Requesting Change

Dye penetrant examinations have been performed on all six reactor vessel nozzles and volumetric examinations have been performed on two of the nozzles with no reportable defects found. The alternate examination methods and frequency proposed by the change will provide sufficient data to detect nozzle-to-safe end defects, minimize the number of core barrel lifts that must be made to perform the examinations and will reduce the personnel radiation exposure related to this task.

### Evaluation

The alternate examinations proposed by the licensee have been evaluated and found to be beyond Section XI requirements. The proposed examinations will reduce personnel exposure to radiation and enhance the effectiveness of the nozzle-to-safe end weld examinations. We therefore find the Technical Specification change acceptable.

The licensee has requested to change Technical Specification Section 4.4.5.2 to read:

...In Article 8 - "Eddy current examination method for installed non-ferromagnetic steam generator heat exchanger tubing," as contained in ASME Boiler and Pressure Vessel Code, Section V - "Nondestructive Examination," and referenced in ASME Boiler and Pressure Vessel Code - Appendix IV of the 1980 Edition through Winter 1980 Addenda of Section XI - "Inservice Inspection of Nuclear Power Plant Components." The tubes ...

### Licensee Basis for Requesting Change

The Technical Specifications presently recognize the ASME Section XI 1974 Edition through the 1975 Summer Addenda, a review of the ASME Section XI Appendix IV Summer 1976 Addenda has been performed and it was found that the newer edition is the first time that inservice inspection of installed steam generator tubing is addressed, therefore, the new edition must be referenced to satisfy the Technical Specification surveillance requirements. The Inservice Inspection program pertaining to steam generator tube inspection will then be revised to comply with the newer edition, Appendix IV of ASME Section XI Summer 1976 Addenda.

### Evaluation

The original request was to update to the Summer 1976 Addenda of the 1974 Edition of Section XI for reasons explained above. The 1980 Edition is more complete in requirements for steam generator tube examination than the Summer 1976 Addenda of the 1974 Edition of Section XI. The 1980 Edition contains

requirements for examination of personnel and references Section V for examination method requirements. The 1980 Edition through Winter 1980 Addenda was recently approved by the NRC and incorporated in paragraph (b) of 10 CFR 50.55a. The licensee agreed to use the 1980 Edition of Section XI for steam generator tube examination in lieu of the 1976 Winter Addenda of the 1974 Edition of Section XI. The staff finds this change acceptable.

#### ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §1.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 16, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-334DUQUESNE LIGHT COMPANYOHIO EDISON COMPANYPENNSYLVANIA POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 48 to Facility Operating License No. DPR-66 issued to Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees), which revised Technical Specifications for operation of the Beaver Valley Power Station, Unit No. 1 (the facility) located in Beaver County, Pennsylvania. The amendment is effective as of the date of issuance.

The amendment revises reactor vessel nozzle-to-safe end weld inspection requirements and updates the ASME code reference for steam generator tube inspections.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

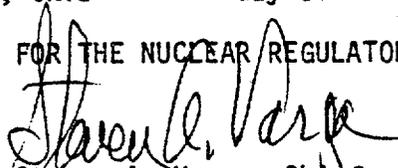
- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §1.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated November 9, 1981, (2) Amendment No. 48 to License No. DPR-66 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the B. F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania 15001. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 16th day of March, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing