MAR 30 1982

Docket No. 50-334

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Mr. J. J. Carey, Vice President
Nuclear Division
Duquesne Light Company
435 Sixth Avenue
Pittsburgh, Pennsylvania 15219

Dear Mr. Carey:

The Commission has issued the enclosed Amendment No. 49 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated February 11, 1982.

The amendment modifies the inservice surveillance requirements for safety related snubbers.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

Dennis A. Chaney, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 49 to DPR-66
2. Safety Evaluation
3. Notice of Issuance

cc w/encl:
See next page

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as to item 3 only + Notice only

OFFICE	DL:ORB#1	DL:ORB#1	DL:ORB#1	DL:OR	OELD	DL:ORB#3	DL:ORAB
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DATE	3/19/82	3/19/82	3/19/82	3/21/82	3/29/82	3/19/82	3/19/82

Mr. J. J. Carey
Duquesne Light Company

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Mr. J. J. Carey
Duquesne Light Company

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 49
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated February 11, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 49 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 30, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 49 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

Remove Pages	Insert Pages
3/4 7-26	3/4 7-26
3/4 7-27	3/4 7-27
3/4 7-28	3/4 7-28
3/4 7-29	3/4 7-29
3/4 7-30	3/4 7-30
3/4 7-31	3/4 7-31
3/4 7-32	3/4 7-32a
3/4 7-32a	3/4 7-32b
3/4 7-32b	3/4 7-32c
3/4 7/32c	3/4 7-32d
3/4 7-32d	3/4 7-32e
3/4 7-32e	3/4 7-32f
--	3/4 7-32g
--	3/4 7-32h
--	3/4 7-32i
3/4 7-33	3/4 7-33
B3/4 7-6	B3/4 7-6
--	B3/4 7-6a
6-19	6-19

3/4.7.12 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.12 All snubbers listed in Tables 3.7-4a and 3.7-4b shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4. (MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES).

ACTION:

With one or more snubbers inoperable, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.12.c on the supported component or declare the supported system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.12 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

a. Visual Inspections

The first inservice visual inspection of snubbers shall be performed after four months but within 10 months of commencing POWER OPERATION and shall include all snubbers listed in Tables 3.7-4a and 3.7-4b. If less than two (2) snubbers are found inoperable during the first inservice visual inspection, the second inservice visual inspection shall be performed 12 months ± 25% from the date of the first inspection. Otherwise, subsequent visual inspections shall be performed in accordance with the following schedule:

No. Inoperable Snubbers per Inspection Period	Subsequent Visual Inspection Period* #
0	18 months ± 25%
1	12 months ± 25%
2	6 months ± 25%
3,4	124 days ± 25%
5,6,7	62 days ± 25%
8 or more	31 days ± 25%

The snubbers may be categorized into two groups: those accessible and those inaccessible during reactor operation. Each group may be inspected independently in accordance with the above schedule.

* The inspection interval shall not be lengthened more than one step at a time.

\# The provisions of Specification 4.0.2 are not applicable.

SURVEILLANCE REQUIREMENTS (Continued)

b. Visual Inspection Acceptance Criteria

Visual inspections shall verify (1) that there are no visible indications of damage or impaired OPERABILITY, (2) attachments to the foundation or supporting structure are secure, and (3) in those locations where snubber movement can be manually induced without disconnecting the snubber, that the snubber has freedom of movement and is not frozen up. Snubbers which appear inoperable as a result of visual inspections may be determined OPERABLE for the purpose of establishing the next visual inspection interval, providing that (1) the cause of the rejection is clearly established and remedied for that particular snubber and for other snubbers that may be generically susceptible; and (2) the affected snubber is functionally tested in the as-found condition and determined OPERABLE per Specification 4.7.12.d or 4.7.12.e, as applicable. However, when the fluid port of a hydraulic snubber is found to be uncovered, the snubber shall be determined inoperable and cannot be determined OPERABLE via functional testing for the purpose of establishing the next visual inspection interval.

c. Functional Tests

At least once per 18 months during shutdown, a representative sample (of at least 10 snubbers or at least 10% whichever is less) of the total of each type of snubber in use in the plant shall be functionally tested either in place or in a bench test. For each snubber that does not meet the functional test acceptance criteria of Specification 4.7.12.d or 4.7.12.e, an additional 10 snubbers or at least 10% whichever is less of that type of snubber shall be functionally tested.

The representative sample selected for functional testing shall include the various configurations, operating environments and the range of size and capacity of snubbers. At least 25% of the snubbers in the representative sample shall include snubbers from the following three categories:

1. The first snubber away from each reactor vessel nozzle

2. Snubbers within 5 feet of heavy equipment (valve, pump, turbine, motor, etc.)

3. Snubbers within 10 feet of the discharge from a safety relief valve.

Snubbers identified in Tables 3.7-4a and 3.7-4b as "Especially Difficult to Remove" or in "High Radiation Zones During Shutdown" shall also be included in the representative sample.* Tables 3.7-4a and 3.7-4b may be used jointly or separately as the basis for the sampling plan.

If a spare snubber has been installed in place of a failed snubber, the spare snubber shall be retested. Test results of this snubber may not be included for the re-sampling.

If any snubber selected for functional testing either fails to lockup or fails to move, i.e., frozen in place, the cause will be evaluated and if caused by manufacturer or design deficiency all snubbers of the same design subject to the same defect shall be functionally tested. This testing requirement shall be independent of the requirements stated above for snubbers not meeting the functional test acceptance criteria.

For the snubber(s) found inoperable, an engineering evaluation shall be performed on the components which are supported by the snubber(s). The purpose of this engineering evaluation shall be to determine if the components supported by the snubber(s) were adversely affected by the inoperability of the snubber(s) in order to ensure that the supported component remains capable of meeting the designed service.

d. Hydraulic Snubbers Functional Test Acceptance Criteria

The hydraulic snubber functional test shall verify that:

1. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.

2. Snubber bleed, or release rate, where required, is within the specified range in compression or tension. For snubbers specifically required to not displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

* Permanent or other exemptions from functional testing for individual snubbers in these categories may be granted by the Commission only if a justifiable basis for exemption is presented and/or snubber life destructive testing was performed to qualify snubber operability for all design conditions at either the completion of their fabrication or at a subsequent date.

SURVEILLANCE REQUIREMENTS (Continued)

e. Mechanical Snubbers Functional Test Acceptance Criteria

The mechanical snubber functional test shall verify that:

1. The force that initiates free movement of the snubber rod in either tension or compression is less than the specified maximum drag force.

2. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.

3. Snubber release rate, where required, is within the specified range in compression or tension. For snubbers specifically required not to displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

f. Snubber Service Life Monitoring

A record of the service life of each snubber, the date at which the designated service life commences and the installation and maintenance records on which the designated service life is based shall be maintained as required by Specification 6.10.2.m.

Concurrent with the first inservice visual inspection and at least once per 18 months thereafter, the installation and maintenance records for each snubber listed in Tables 3.7-4a and 3.7-4b shall be reviewed to verify that the indicated service life has not been exceeded or will not be exceeded prior to the next scheduled snubber service life review. If the indicated service life will be exceeded prior to the next scheduled snubber service life review, the snubber service life shall be reevaluated or the snubber shall be replaced or reconditioned so as to extend its service life beyond the date of the next scheduled service life review. This reevaluation, replacement or reconditioning shall be indicated in the records.

TABLE 3.7-4a

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION				ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
RC-HC-1A	RC	RCP	Cub. A	736'6"	I	Yes	Yes
RC-HC-2A	RC	RCP	Cub. A	731'2"	"	"	"
RC-HC-3A	"	"	"	736'6"	"	"	"
RC-HC-4A	"	"	"	736'6"	"	"	"
RC-HC-5A	"	"	"	728'2"	"	"	"
RC-HC-6A	"	"	"	736'6"	"	"	"
RC-HC-7A	"	"	"	728'2"	"	"	"
RC-HC-8A	"	"	"	731'2"	"	"	"
RC-HC-9A	"	"	"	766'10"	"	"	"
RC-HC-10A	"	"	"	766'10"	"	"	"
RC-HC-11A	"	"	"	766'10"	"	"	"
RC-HC-12A	"	"	"	766'10"	"	"	"
RC-HC-1B	RC	RCP	Cub. B	728'2"	"	"	"
RC-HC-2B	"	"	"	728'2"	"	"	"
RC-HC-3B	"	"	"	731'2"	"	"	"
RC-HC-4B	"	"	"	736'6"	"	"	"
RC-HC-5B	"	"	"	736'6"	"	"	"
RC-HC-6B	"	"	"	736'6"	"	"	"
RC-HC-7B	"	"	"	736'6"	"	"	"
RC-HC-8B	"	"	"	731'2"	"	"	"
RC-HC-9B	"	"	"	766'10"	"	"	"
RC-HC-10B	"	"	"	766'10"	"	"	"
RC-HC-11B	"	"	"	766'10"	"	"	"
RC-HC-12B	"	"	"	766"10"	"	"	"

*Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4 provided that safety evaluations, documentation and reporting are provided in accordance with 10 CFR 50.59 and that a revision to Table 3.7-4 is included with a subsequent License Amendment request.

TABLE 3.7-4a (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION				ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
RC-HC-1C	RC	RCP	Cub. C	731'2"	I	Yes	Yes
RC-HC-2C	"	"	"	736'2"	"	"	"
RC-HC-3C	"	"	"	728'2"	"	"	"
RC-HC-4C	"	"	"	736'6"	"	"	"
RC-HC-5C	"	"	"	736'6"	"	"	"
RC-HC-6C	"	"	"	731'2"	"	"	"
RC-HC-7C	"	"	"	728'2"	"	"	"
RC-HC-8C	"	"	"	736'6"	"	"	"
RC-HC-9C	"	"	"	766'10"	"	"	"
RC-HC-10C	"	"	"	766'10"	"	"	"
RC-HC-11C	"	"	"	766'10"	"	"	"
RC-HC-12C	"	"	"	766'10"	"	"	"
RC-HSS-5	RC	REA.	CONT.BLDG.	716'	I	Yes	Yes
RC-HSS-6	"		"	716'	"	"	"
RC-HSS-7	"		"	716'	"	"	"
RC-HSS-8	"		"	716'	"	"	"
RC-HSS-9	"		"	716'	"	"	"
RC-HSS-10	"		"	716'	"	"	"
RH-HSS-123	RHR		"	699'	A	No	No
RH-HSS-124	"		"	699'	"	"	"
SI-HSS-110	SI		"	708'	"	"	"
SI-HSS-111	"		"	708'	"	"	"
SI-HSS-119	"			697'	"	"	"
SI-HSS-120	"		"	697'	"	"	"
SI-HSS-108	"		"	708'	"	"	"

*Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4 provided that safety evaluations, documentation and reporting are provided in accordance with 10 CFR 50.59 and that a revision to Table 3.7-4 is included with a subsequent License Amendment request.

TABLE 3.7-4a (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION				ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
SI-HSS-103	SI	REA.	CONT. BLDG.	695'	A	No	No
SI-HSS-101	"		"	705'	"	"	"
SI-HSS-104	"		"	705'	"	"	"
RH-HSS-106	RHR		"	704'	"	Yes	"
RH-HSS-109	"		"	704'	"	"	"
RH-HSS-110	"		"	704'	"	"	"
RH-HSS-112	"		"	704'	"	"	"
RC-HSS-11	RC	RCP	Cub. A	723'	I	Yes	Yes
RC-HSS-12	"	"	"	723'	"	"	"
RC-HSS-13	"	"	"	735'	"	"	"
RC-HSS-14	"	"	"	735'	"	"	"
SI-HSS-412	SI	RCP	Cub. A	732'	"	"	"
SI-HSS-413	"	"	"	732'	"	"	"
SI-HSS-415	"	"	"	738'	"	"	"
PS-243-VS-1	RC	"	"	725'	"	"	"
VS-357-1	"	"	"	719'	"	"	"
SI-HSS-416	SI	RCP	Cub. B	730'	"	"	"
SI-HSS-417	"	"	"	730'	"	"	"
SI-HSS-419	"	"	"	738'	"	"	"
SI-HSS-117	"	"	"	737'	"	"	"
SI-HSS-118	"	"	"	737'	"	"	"
PS-244-VS-1	RC	"	"	725'	"	"	"
VS-358-1	"	"	"	719'	"	"	"
RC-HSS-1	RC	RCP	Cub. C	734'	I	Yes	Yes
RC-HSS-2	"	"	"	734'	"	"	"

*Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4 provided that safety evaluations, documentation and reporting are provided in accordance with 10 CFR 50.59 and that a revision to Table 3.7-4 is included with a subsequent License Amendment request.

TABLE 3.7-4a (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION				ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
RC-HSS-3	RC	RCP	Cub. C	722'	I	Yes	Yes
RC-HSS-4	"	"	"	722'	"	"	"
PS-245-VS-1	"	"	"	725'	"	"	"
VS-359-1	"	"	"	719'	"	"	"
SI-HSS-420	SI	"	"	731'	"	"	"
SI-HSS-421	"	"	"	731'	"	"	"
RC-HSS-101	RC	RCP	Cub. A.	739'	I	Yes	Yes
RC-HSS-102	"	"	"	739'	"	"	"
SI-HSS-102A	SI	"	"	745'	"	"	"
SI-HSS-102B	"	"	"	745'	"	"	"
SI-HSS-414	"	"	"	741'	"	"	"
RC-HSS-103	RC	RCP	Cub. B	739'	"	"	"
RC-HSS-104	"	"	"	739'	"	"	"
SI-HSS-418	SI	"	"	741'	"	"	"
RC-HSS-23	RC	REA. CONT. BLDG.		749'	A	No	No
RC-HSS-105	RC	RCP	Cub. C	739'	I	Yes	Yes
RC-HSS-106	RC	"	"	739'	"	"	"
RC-HSS-130	"	"	"	739'	I	"	"
RC-HSS-131	"	"	"	739'	"	"	"
SI-HSS-114A	SI	"	"	745'	"	"	"
SI-HSS-114B	"	"	"	745'	"	"	"
SI-HSS-422	"	"	"	741'	"	"	"
SI-HSS-423	"	"	"	739'	"	"	"
RC-HSS-22	RC	Pressurizer Cub.		784'	A	Yes	Yes
RC-HSS-41A	"	"		784'	"	"	"

*Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4 provided that safety evaluations, documentation and reporting are provided in accordance with 10 CFR 50.59 and that a revision to Table 3.7-4 is included with a subsequent License Amendment request.

TABLE 3.7-4a (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION			ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
RC-HSS-44A	RC	Pressurizer Cub.	784'	A	Yes	Yes
RC-HSS-302	"	"	784'	"	"	"
RC-HSS-303A	"	"	784'	"	"	"
RC-HSS-303B	"	"	784'	"	"	"
RS-HSS-235	RS	REA. CONT. BLDG.	832'	I	No	"
RS-HSS-210	"	"	817'	"	"	"
RS-HSS-211	"	"	817'	"	"	"
RS-HSS-213	"	"	800'	"	"	"
RS-HSS-214	"	"	800'	"	"	"
RS-HSS-222	"	"	814'	"	"	"
RS-HSS-223	"	"	814'	"	"	"
RS-HSS-224	"	"	813'	"	"	"
RS-HSS-225	"	"	813'	"	"	"
WFPD-HSS-201	FW	"	780'	A	"	No
WFPD-HSS-202	"	"	780'	"	"	"
SHP-HSS-201	MS	"	805'	I	"	Yes
SHP-HSS-202	"	"	805'	"	"	"
SHP-HSS-203	"	"	805'	"	"	"
SHP-HSS-204	"	"	805'	"	"	"
WFPD-HSS-203	FW	"	780'	A	"	No
WFPD-HSS-204	"	"	780'	"	"	"
WFPD-HSS-205	"	"	780'	"	"	"
WFPD-HSS-206	"	"	780'	"	"	"
SHP-HSS-205	MS	"	798'	I	"	Yes
SHP-HSS-206	"	"	798'	"	"	"

*Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4 provided that safety evaluations, documentation and reporting are provided in accordance with 10 CFR 50.59 and that a revision to Table 3.7-4 is included with a subsequent License Amendment request.

TABLE 3.7-4a (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION			ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
WFPD-HSS-207	FW	REA. CONT. BLDG.	772'	A	No	No
WFPD-HSS-208	"	"	772'	"	"	"
WFPD-HSS-208A	"	"	771'	"	"	"
SHP-HSS-207	MS	"	796'	I	"	Yes
SHP-HSS-208	"	"	796'	"	"	"
WFPD-HSS-211	FW	"	772'	A	"	No
WFPD-HSS-212	"	"	772'	"	"	"
WFPD-HSS-212A	"	"	771'	"	"	"
SHP-HSS-213	MS	"	798'	I	"	Yes
SHP-HSS-214	"	"	798'	"	"	"
SHP-HSS-209	"	"	798'	"	"	"
SHP-HSS-210	"	"	798'	"	"	"
SHP-HSS-211	"	"	798'	"	"	"
SHP-HSS-212	"	"	798'	"	"	"
SHP-HSS-214A	"	"	805'	"	"	"
WFPD-HSS-209	FW	"	780'	A	"	No
WFPD-HSS-210	"	"	780'	"	"	"
RC-HSS-128	RC	"	734'	"	Yes	"
RC-HSS-129	"	"	734'	"	"	"
RC-HSS-119	"	"	734'	"	"	"
SI-PSSP-337	SI	"	728'	"	No	"
SI-HSS-409	"	"	729'	"	"	"
SI-HSS-410	"	"	731'	"	"	"
SI-HSS-411	"	"	731'	"	"	"
RS-HSS-201	RS	"	731'	"	Yes	"

*Snubbers may be added to safety related systems without prior License Admendment to Table 3.7-4 provided that safety evaluations, documentation and reporting are provided in accordance with 10CFR 50.59 and that a revision to Table 3.7-4 is included with a subsequent License Amendment request.

TABLE 3.7-4a (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION			ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
RS-HSS-202	RS	REA. CONT. BLDG.	731'	A	Yes	No
RS-HSS-237	"	"	731'	"	No	"
RS-HSS-238	"	"	731'	"	"	"
RS-HSS-229	"	"	731'	"	"	"
RS-HSS-236	"	"	731'	"	"	"
RS-HSS-234	"	"	726'	"	"	"
CC-HSS-405A	"	"	707'	I	Yes	Yes
CC-HSS-405B	"	"	707'	"	"	"
CC-HSS-407A	"	"	711'	"	"	"
CC-HSS-407B	"	"	711'	"	"	"
RS-HSS-205	RS	"	702'	A	No	No
RS-HSS-206	"	"	702'	"	"	"
RS-HSS-219	"	"	702'	"	"	"
RS-HSS-220	"	"	702'	"	"	"
RS-HSS-207	"	"	702'	"	"	"
RS-HSS-208	"	"	702'	"	"	"
RS-HSS-209	"	"	710'	"	"	"
RS-HSS-215	"	"	715'	"	"	"
RS-HSS-216	"	"	715'	"	"	"
RH-HSS-105	RH	"	704'	I	Yes	Yes
RH-HSS-107	"	"	704'	"	"	"
RH-HSS-108	"	"	704'	"	"	"
RH-HSS-111	"	"	704'	"	"	"
RH-HSS-113	"	"	710'	"	"	"
RH-HSS-114	"	"	710'	"	"	"

*Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4 provided that safety evaluations, documentation and reporting are provided in accordance with 10 CFR 50.59 and that a revision to Table 3.7-4 is included with a subsequent License Amendment request.

TABLE 3.7-4a (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION			ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
CC-HSS-404	CC	REA. CONT. BLDG.	711'	A	Yes	No
CC-HSS-408	"	"	711'	I	"	"
RH-HSS-101	RH	"	704'	A	No	"
RH-HSS-102	"	"	704'	A	"	"
RH-HSS-402	"	"	708'	"	"	"
RH-HSS-403	"	"	708'	"	"	"
RH-HSS-121	"	"	699'	"	"	"
RH-HSS-122	"	"	699'	"	"	"
RS-HSS-203	RS	"	701'	"	"	"
RS-HSS-221	"	"	702'	"	"	"
CC-HSS-401	CC	P.A. BLDG.	736'	"	"	"
CC-HSS-402	"	"	736'	"	"	"
CC-HSS-403	"	"	736'	"	"	"
CC-HSS-404	"	"	745'	"	Yes	"
CC-HSS-405	"	"	742'	"	No	"
CC-PSSP-301	"	"	724'	"	"	"
WR-HSS-310	RW	"	731'	"	"	"
CH-HSS-305	CH	"	720'	"	"	"
SI-HSS-309	SI	"	720'	I	"	Yes
CH-HSS-311	CH	"	741'	"	Yes	"
WR-HSS-311	RW	"	724'	A	No	No
WR-HSS-312	"	"	724'	"	"	"
WR-HSS-313	"	"	724'	"	"	"
WR-HSS-314	"	"	731'	"	"	"
WR-HSS-301	"	CABLE VAULT	730'	"	"	"
WR-HSS-302	"	"	730'	"	"	"

*Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4 provided that safety evaluations, documentation and reporting are provided in accordance with 10 CFR 50.59 and that a revision to Table 3.7-4 is included with a subsequent License Amendment request.

TABLE 3.7-4a (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION		ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
WR-HSS-303A	RW CABLE VAULT	731'	A	No	No
WR-HSS-303B	" "	731'	"	"	"
WR-HSS-304A	" "	728'	"	"	"
WR-HSS-304B	" "	728'	"	"	"
WR-HSS-316	" "	733'	"	"	"
WR-HSS-306	" "	733'	"	"	"
WR-HSS-308	" "	731'	"	"	"
WR-HSS-309	" "	731'	"	"	"
WR-HSS-300	" "	724'	"	"	"
SI-HSS-522	SI "	731'	"	"	"
SI-HSS-523A	" "	731'	"	"	"
SI-HSS-523B	" "	731'	"	"	"
SI-HSS-521	" "	731'	"	"	"
SI-HSS-516A	" "	731'	"	"	"
SI-HSS-516B	" "	731'	"	"	"
SI-HSS-520	" "	731'	"	"	"
SI-HSS-515	" "	731'	"	"	"
SI-HSS-519	" "	731'	"	"	"
SI-HSS-514	" "	731'	"	"	"
SI-HSS-512	" "	738'	"	"	"
SI-HSS-512A	" "	734'	"	"	"
SI-HSS-511	" "	738'	"	"	"
SI-HSS-518	" "	733'	"	"	"
SI-HSS-517	" "	738	"	"	"
QS-HSS-504	QS SAFEGUARDS AREA	741'	"	"	"

*Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4 provided that safety evaluations, documentation and reporting are provided in accordance with 10 CFR 50.59 and that a revision to Table 3.7-4 is indluded with a subsequent License Amendment request.

TABLE 3.7-4a (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION			ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
QS-PSSP-501	QS	SAFEGUARDS AREA	734'	A	No	No
SI-PSSP-002	SI	"	743'	"	"	"
SI-PSSP-003	"	"	743'	"	"	"
SI-PSSP-009	"	"	743'	"	"	"
SI-PSSP-010	"	"	743'	"	"	"
SI-PSSP-067C	"	"	743'	"	"	"
SI-PSSP-060D	"	"	743'	"	"	"
SI-PSSP-033A	"	"	753'	"	"	"
SI-PSSP-036A	"	"	753'	"	"	"
QS-HSS-205A	QS	"	735'	"	"	"
QS-HSS-205B	"	"	735'	"	"	"
QS-HSS-202	"	"	737'	"	"	"
SI-HSS-212A	SI	"	737'	"	"	"
SI-HSS-212B	"	"	734'	"	"	"
RS-HSS-210A	RS	SAFEGUARDS AREA	736'	"	"	"
RS-HSS-209A	"	"	736'	"	"	"
RH-HSS-203	RH	"	736'	"	"	"
RH-HSS-204A	"	"	736'	"	"	"
RH-HSS-204B	"	"	736'	"	"	"

*Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4 provided that safety evaluations, documentation and reporting are provided in accordance with 10 CFR 50.59 and that a revision to Table 3.7-4 is included with a subsequent License Amendment request.

TABLE 3.7-4a (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION			ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE	ESPECIALLY DIFFICULT TO REMOVE
FC-HSS-201	FC	FUEL BUILDING	747'	A	No	No
FC-HSS-5A	"	"	747'	"	"	"
FC-HSS-202	"	"	745'	"	"	"
FC-HSS-203	"	"	745'	"	"	"
FC-HSS-204	"	"	742'	"	"	"
FC-HSS-205	"	"	742'	"	"	"
FC-HSS-206	"	"	742'	"	"	"

*Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4 provided that safety evaluations, documentation and reporting are provided in accordance with 10 CFR 50.59 and that a revision to Table 3.7-4 is included with a subsequent License Amendment request.

TABLE 3.7-4b

SAFETY RELATED MECHANICAL SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION			ACCESSIBLE OR INACCESSIBLE	HIGH RADIATION ZONE **	ESPECIALLY DIFFICULT TO REMOVE
RC-PSSP-301	RC	Pressurizer Cub.	784'	I	Yes	Yes
CC-PSSP-300A	CC	P.A. BLDG.	726'	A	No	No
CC-PSSP-300B	CC	"	726'	"	"	"
CC-HSS-406A	CC	Pipe Annulus	710'	I	Yes	Yes
CC-HSS-406B	CC	"	710'	"	"	"
WR-HSS-307	RW	P.A. BLDG.	732'	A	No	No

* Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-4b provided that a revision to Table 3.7-4b is included with the next License Amendment request.

**Modifications to this column due to changes in high radiation areas may be made without prior License Amendment provided that a revision to Table 3.7-4b is included with the next License Amendment request.

3/4.7.12 SNUBBERS

All snubbers are required OPERABLE to ensure that the structural integrity of the reactor coolant system and all other safety-related systems is maintained during and following a seismic or other similar event initiating dynamic loads. Snubbers excluded from this inspection program are those installed on nonsafety-related systems and then only if their failure or failure of the system on which they are installed, would have no adverse effect on any safety-related system.

The visual inspection frequency is based upon maintaining a constant level of snubber protection to systems. Therefore, the required inspection interval varies inversely with the observed snubber failures and is determined by the number of inoperable snubbers found during an inspection. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection.

When the cause of the rejection of a snubber is clearly established and remedied for that snubber and for any other snubbers that may be generically susceptible, and verified by inservice functional testing, that snubber may be exempted from being counted as inoperable. Generically susceptible snubbers are those which are of a specific make or model and have the same design features directly related to rejection of the snubber by visual inspection, or are similarly located or exposed to the same environmental conditions such as temperature, radiation and vibration.

When a snubber is found inoperable, an engineering evaluation is performed, in addition to the determination of the snubber mode of failure, in order to determine if any safety-related component or system has been adversely affected by the inoperability of the snubber. The engineering evaluation shall determine whether or not the snubber mode of failure has imparted a significant effect or degradation on the supported component or system.

To provide assurance of snubber functional reliability, a representative sample of the installed snubbers will be functionally tested during plant shutdowns at refueling or 18 month intervals not to exceed two (2) years. Observed failures of these sample snubbers shall require functional testing of additional units.

Hydraulic snubbers and mechanical snubbers may each be treated as a different entity for the above surveillance programs.

BASES

The service life of a snubber is evaluated via manufacturer input and information through consideration of the snubber service conditions and associated installation and maintenance records (newly installed snubber, seal replaced, spring replaced, in high radiation area, in high temperature area, etc. . .). The requirement to monitor the snubber service life is included to ensure that the snubbers periodically undergo a performance evaluation in view of their age and operating conditions. These records will provide statistical bases for future consideration of snubber service life. The requirements for the maintenance of records and the snubber service life review are not intended to affect plant operation.

j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

k. Records of meetings of the OSC and the ORC.

l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.13.

m. Records of the service lives of all hydraulic and mechanical snubbers listed on Tables 3.7-4a and 3.7-4b including the date at which the service life commences and associated installation and maintenance records.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20:

a. Each High Radiation Area in which the intesity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.12.1a above, and in addition locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift Supervisor on duty.

6.13 ENVIRONMENTAL QUALIFICATION

6.13.1 By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of: Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class 1E Electrical Equipment in Operating Reactors" (DOR Guidelines); or, NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment," December 1979. Copies of these documents are attached to Order for Modification of License No. DPR-66 dated October 24, 1980.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 49 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

Introduction

To reflect accumulated experience obtained from operating plants in the past several years, NRC issued Revision 1 of the Standard Technical Specifications on the surveillance requirements for safety-related snubbers. On November 20, 1980, this document was transmitted to operating plants excluding those under SEP along with a request for submittal of appropriate license amendments to incorporate the requirements of this revision within 120 days. The same request was extended to SEP plants on March 23, 1981.

Description and Discussion

Numerous discoveries of inoperative snubbers in the period of 1973 to 1975 resulted in their surveillance requirements in the Technical Specifications for operating reactor plants. However, several deficiencies were identified after the original requirements were in force for several years. These deficiencies are:

1. Mechanical snubbers were not included in these requirements.

2. The rated capacity of snubbers was used as a limit to the inservice test requirement.

3. NRC approval was necessary for the acceptance of seal materials.

4. Inservice test requirements were not clearly defined.

5. In-place inservice testing was not permitted.

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Since mechanical snubbers were not subject to any surveillance requirements, some licensees and permit holders believed that mechanical snubbers were preferred by NRC. Many plants used mechanical snubbers as original equipment and many others requested to replace their hydraulic snubbers with mechanical ones to simplify or avoid an inservice surveillance program. This is directly contradictory to NRC's intention, where for an unsurveyed mechanical snubber, the most likely failure is permanent lock-up. This failure mode can be harmful to the system during normal plant operations.

1. Mechanical snubbers are now included in the surveillance program.

2. No arbitrary snubber capacity is used as a limit to the inservice test requirements.

3. Seal material no longer requires NRC approval. A monitoring program shall be implemented to assure that snubbers are functioning within their service life.

4. Clearly defined inservice test requirements for snubbers shall be implemented.

5. In-place inservice testing shall be permitted.

Evaluation

The proposed license amendment submitted by Duquesne Light Company for Operating License DPR-66 for Beaver Valley Power Station, Unit No. 1 has incorporated the necessary requirements in Revision 1 of the Standard Technical Specifications for the surveillance of safety-related snubbers and is therefore, acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 30, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-334

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 49 to Facility Operating License No. DPR-66 issued to Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees), which revised Technical Specifications for operation of the Beaver Valley Power Station, Unit No. 1 (the facility) located in Beaver County, Pennsylvania. The amendment is effective as of the date of issuance.

The amendment modifies the inservice surveillance requirements for safety related snubbers.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to

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10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated February 11, 1982, (2) Amendment No. 49 to License No. DPR-66 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C. and at the B. F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania 15001. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 30th day of March, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing