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Docket No. 50-334

Mr. J. J. Carey, Vice President
Nuclear Division
Duquesne Light Company
435 Sixth Avenue
Pittsburgh, Pennsylvania 15219



Dear Mr. Carey:

The Commission has issued the enclosed Amendment No. 50 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated November 9, 1981.

The amendment changes the allowable test time for solid state protection system instrumentation channels and the purge and exhaust valve isolation requirements.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

Dennis A. Chaney, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

- 1. Amendment No. 50 to DPR-66
- 2. Safety Evaluation
- 3. Notice of Issuance

cc w/enclosures:
See next page

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DATE	03/19/82	03/19/82	03/19/82	03/23/82	03/31/82		

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Mr. J. J. Carey
Duquesne Light Company

cc: Mr. H. P. Williams
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Duquesne Light Company
Beaver Valley Power Station
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Mr. J. J. Carey
Duquesne Light Company

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Dr. Clark Hansbarger
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Health Department
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 50
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated November 9, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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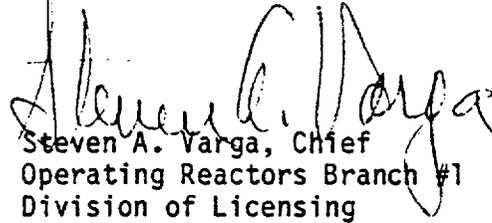
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 50, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 5, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 3-20	3/4 3-20
3/4 6-19c	3/4 6-19c
3/4 9-4	3/4 9-4
3/4 9-9	3/4 9-9

TABLE 3.3-3 (Continued)

TABLE NOTATION

Trip function may be bypassed in this MODE below P-11.

The channel(s) associated with the protective functions derived from the out of service Reactor Coolant Loop shall be placed in the tripped mode.

ACTION STATEMENTS

ACTION 13 - With the number of OPERABLE Channels one less than the Total Number of Channels, be in HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours; however, one channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1.1., provided the other channel is operable.

ACTION 14 - With the number of OPERABLE Channels one less than the Total Number of Channels:

- a. Below P-11 or P-12, place the inoperable channel in the tripped condition within 1 hour; restore the inoperable channel to OPERABLE status within 24 hours after exceeding P-11 or P-12; otherwise be in at least HOT STANDBY within the following 6 hours.
- b. Above P-11 and P-12, place the inoperable channel in the tripped condition within 1 hour; operation may continue until performance of the next required CHANNEL FUNCTIONAL TEST.

ACTION 15 - With a channel associated with an operating loop inoperable, restore the inoperable channel to OPERABLE status within 2 hours or be in HOT SHUTDOWN within the following 12 hours; however, one channel associated with an operating loop may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1.1.

ACTION 16 - With the number of OPERABLE Channels one less than the Total Number of Channels:

- a. Below P-11 or P-12, place the inoperable channel in the bypass condition; restore the inoperable channel to OPERABLE status within 24 hours after exceeding P-11 or P-12; otherwise be in at least HOT SHUTDOWN within the following 12 hours.

TABLE 3.6-1 (Continued)

	VALVE NUMBER		FUNCTION	TESTABLE DURING PLANT OPERATION	ISOLATION TIME (Sec)	
	INSIDE	OUTSIDE			INSIDE	OUTSIDE
C. Containment Purge and Exhaust						
1.	VS-D-5-3R	VS-D-5-3A	Containment Purge Exhaust	No	8 (1)	8 (1)
2.	VS-D-5-5B	VS-D-5-5A	Containment Purge Supply	No	11 (1)	8 (1)
3.		VS-D-5-6	Containment Purge Vacuum Breaker	No	N/A	N/A
D. Manual						
1.	ICH-181	MOV-CH308A	Seal Injection Water to RC Pump	No	N/A	N/A
2.	ICH-182	MOV-CH308B	Seal Injection Water to RC Pump	No	N/A	N/A
3.	ICH-183	MOV-CH308C	Seal Injection Water to RC Pump	No	N/A	N/A
*4.	MOV-CH142		Reactor Coolant System Letdown	Yes	N/A	N/A
*5.	MOV-CC112B3	1CCR-252	CCW from RHR Hx & RHR Pump Seal Coolers	Yes	N/A	N/A
*6.	MOV-CC112A3	1CCR-251	CCW from RHR Hx & RHR Pump Seal Coolers	Yes	N/A	N/A
*7.	MOV-CC112A2	1CCR-247	CCW to RHR Hx & RHR Pump Seal Coolers	Yes	N/A	N/A
*8.	MOV-CC112B2	1CCR-248	CCW to RHR Hx & RHP Pump Seal Coolers	Yes	N/A	N/A
#9.		MOV-FW-151A	Auxiliary Feedwater	Yes	N/A	N/A
#10.		MOV-FW-151B	Auxiliary Feedwater	Yes	N/A	N/A
#11.		MOV-FW-151C	Auxiliary Feedwater	Yes	N/A	N/A
#12.		MOV-FW-151D	Auxiliary Feedwater	Yes	N/A	N/A
#13.		MOV-FW-151E	Auxiliary Feedwater	Yes	N/A	N/A
#14.		MOV-FW-151F	Auxiliary Feedwater	Yes	N/A	N/A
#15.		MOV-RW104A	Riverwater to Recirc. Spray Hx	Yes	N/A	N/A
#16.		MOV-RW104C	Riverwater to Recirc. Spray Hx	Yes	N/A	N/A
#17.		MOV-RW104B	Riverwater to Recirc. Spray HX	Yes	N/A	N/A
#18.		MOV-RW104D	Riverwater to Recirc. Spray Hx	Yes	N/A	N/A

(1) APPLICABILITY: During CORE ALTERATIONS or movement of irradiated fuel within the containment. The provisions of specification 3.0.4 are not applicable. The Containment Purge Exhaust and Supply valves will be locked shut during operation in modes 1, 2, 3, and 4.

REFUELING OPERATION

CONTAINMENT BUILDING PENETRATIONS

LIMITING CONDITION FOR OPERATION

3.9.4 The containment building penetrations shall be in the following status:

- a. The equipment door closed and held in place by a minimum of four bolts,
- b. A minimum of one door in each airlock is closed, and
- c. Each penetration providing direct access from the containment atmosphere to the outside atmosphere shall be either:
 1. Closed by an isolation valve, blind flange, or manual valve, or
 2. Exhausting at less than or equal to 7500 cfm through OPERABLE Containment Purge and Exhaust Isolation Valves with isolation times as specified in Table 3.6-1 to OPERABLE HEPA filters and charcoal adsorbers of the Supplemental Leak Collection and Release System (SLCRS).

APPLICABILITY: During CORE ALTERATIONS or movement of irradiated fuel within the containment.

ACTION:

With the requirements of the above specification not satisfied, immediately suspend all operations involving CORE ALTERATIONS or movement of irradiated fuel in the containment. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.4.1 Each of the above required containment penetrations shall be determined to be in its above required condition within 150 hours prior to the start of and at least once per 7 days during CORE ALTERATIONS or movement of irradiated fuel in the containment.

4.9.4.2 The containment purge and exhaust system shall be demonstrated OPERABLE by:

- a. Verifying the flow rate through the SLCRS at least once per 24 hours when the system is in operation.
- b. Testing the Containment Purge and Exhaust Isolation Valves per the applicable portions of Specification 4.6.3.1.2, and
- c. Testing the SLCRS per Specification 4.7.8.1.

REFUELING OPERATIONS

CONTAINMENT PURGE AND EXHAUST ISOLATION SYSTEM

LIMITING CONDITION FOR OPERATION

3.9.9 The Containment Purge and Exhaust isolation system shall be OPERABLE.

APPLICABILITY: During CORE ALTERATIONS or movement of irradiated fuel within the containment.

ACTION:

With the Containment Purge and Exhaust isolation system inoperable, close each of the purge and exhaust penetrations providing direct access from the containment atmosphere to the outside atmosphere. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.9 The Containment Purge and Exhaust isolation system shall be demonstrated OPERABLE within 150 hours prior to the start of and at least once per 7 days during CORE ALTERATIONS by verifying that containment Purge and Exhaust isolation occurs on manual initiation and on a high-high radiation signal from each of the containment radiation monitoring instrumentation channels.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

Background

By letter dated November 9, 1981, Duquesne Light Company (the licensee) proposed five revisions to the Technical Specifications of Beaver Valley Power Station, Unit No. 1. Two of these proposed changes were evaluated in a Safety Evaluation issued March 16, 1982, the remaining three are evaluated herein.

Item 1

The technical specifications require that the Solid State Protection System along with its instrumentation channels, interlocks and the automatic logic and relays be operable. Presently the technical specifications allow one channel to be bypassed for up to one hour while surveillance testing is being performed.

The licensee has proposed that the time allowed for surveillance testing be expanded to two hours.

Item 2

Periodic leak rate testing of the containment is required by Appendix J to 10 CFR Part 50. Presently the technical specifications references the testing methods of ANSI N45.4-1972, "Leakage Rate Testing of Containment Structures for Nuclear Reactors."

The licensee has proposed referencing ANSI N56.8-1981, "Containment System Leakage Testing Requirements," and deleting the reference to ANSI N45.4-1972 for their leak testing methods.

Item 3

The containment purge and exhaust isolation valves are required to be locked closed during operating modes 1, 2, 3, and 4. During operating modes 5 and 6 the technical specifications presently require that the isolation valves be operable.

The licensee has proposed that the stroke times for the isolation valves only be applicable during mode 6 operation.

EVALUATION

Item 1

The present technical specifications allow bypassing of a Solid State Protection System instrumentation channel for up to one hour while surveillance testing is being performed. If the surveillance testing cannot be completed in this time frame, the plant must begin shutting down and be in Hot Shutdown within 12 hours. The licensee has requested that the Action Statements 13 and 15 accompanying Table 3.3-3 of the plant's technical specifications be revised to allow bypassing of a channel for up to two hours while surveillance testing is being performed. The additional time is being requested to allow sufficient time for testing without running the risk of shutting the plant down.

The NRC staff is in agreement that one hour may not allow sufficient time for testing and that two hours would be a more appropriate time. In September 1981 the NRC standard technical specifications for Westinghouse plants (W-STs) were revised to allow bypassing of instrumentation channels for up to two hours while surveillance testing is being performed. Therefore, we find the licensee's proposal acceptable since it has already been found acceptable and incorporated in the W-STs. As shown in the attachment (page 3/4 3-20), the licensee has agreed to add the phrase, "provided the other channel is OPERABLE" to be in compliance with the wording of the W-STs.

Item 2

ANSI N45.4-1972, "Leakage Rate Testing of Containment Structures for Nuclear Reactors," is referenced in Appendix J to 10 CFR Part 50 as representing acceptable methods of performing required leak rate tests. Similarly, the Beaver Valley Unit 1 technical specifications state that the leak tests will be performed using the methods and provisions of ANSI N45.4-1972.

The licensee has proposed deleting the references to ANSI N45.4-1972 and replacing it with ANSI N56.8-1981, "Containment System Leakage Testing Requirements." The licensee has requested this because ANSI N56.8-1981 permits testing of containment penetrations using water, instead of gas, as the pressurized fluid for Type C leak testing.

There are a number of differences in the leak testing methods between ANSI N45.4-1972 and ANSI N56.8-1981. The provision to use water for the test fluid for Type C leak rate tests is only one such difference.

The staff has previously reviewed ANSI N56.8-1981 and has chosen not to endorse the standard because of the differences in leak testing methods. Therefore we find the licensee's proposal to be unacceptable.

However, with regard to using water as the test fluid for Type C leak testing, the staff has found this practice to be acceptable on a qualified, case-by-case basis. The NRC's Office of Nuclear Regulatory Research is currently in the process of revising Appendix J to define the criteria which will allow Type C leak testing using water as the test fluid.

We have informed the licensee that a blanket reference to ANSI N56.8-1981 is unacceptable. However, we have also informed the licensee that Type C leak testing using water as the test fluid can be found acceptable if certain conditions are met. The licensee has agreed to submit an itemized list of the containment penetrations to be tested in this manner. The staff will review this list upon its submittal.

Item 3

The containment purge and exhaust isolation valves are required to be locked closed during operating modes 1, 2, 3, and 4. The current technical specifications require that the purge and exhaust system isolation valves be operable during operating modes 5 and 6 and capable of automatic isolation with isolation times as specified in Table 3.6-1. Operability during modes 5 and 6 is intended to provide protection against fuel handling accidents.

The licensee's present technical specifications do not allow any time for the isolation valves to be out of service for maintenance or inspection. The valves are required to be locked closed during modes 1, 2, 3, and 4 and OPERABLE during modes 5 and 6. If the valves are found to be inoperable during modes 5 and 6, they are then required to be closed.

The licensee has proposed that the stroke times for the isolation valves only be applicable during mode 6 operation. However, after discussions with the staff, the licensee has agreed to incorporate the wording of the W-STs. As shown in the attachment (pages 3/4 6-19c and 3/4 9-9c) operability of the purge and exhaust system isolation valves will only be required during core alterations or movement of irradiated fuel. We believe these changes will meet their objective of protecting against a fuel handling accident. Therefore, we find these changes acceptable because it has been previously found acceptable and incorporated into the W-STs.

The proposed technical specification changes regarding Solid State Protection System instrumentation channels and the purge and exhaust system isolation valves have been found to be acceptable primarily because they are now consistent with that found in the W-STs.

The proposed change to delete the reference to ANSI N45.4-1972 and insert ANSI N56.8-1981 was not found acceptable. The basis is that the staff review of ANSI N56.8-1981 did not result in an endorsement of the standard. However, as stated previously, the licensee plans to submit a separate proposal for alternative means to Type C leak testing. The staff will review this item when it is received.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 5, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-334DUQUESNE LIGHT COMPANYOHIO EDISON COMPANYPENNSYLVANIA POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 50 to Facility Operating License No. DPR-66 issued to Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees), which revised Technical Specifications for operation of Beaver Valley Power Station, Unit No. 1 (the facility) located in Beaver County, Pennsylvania. The amendment is effective as of the date of issuance.

The amendment changes the allowable test time for solid state protection system instrumentation channels and the purge and exhaust valve isolation requirements.

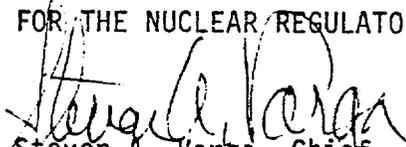
The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated November 9, 1981, (2) Amendment No. 50 to License No. DPR-66 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the B. F. Jones Memorial Library, 663 Franklin Avenue, Aliquippa, Pennsylvania 15001. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 5th day of April, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing