

Mr. James W. Langenbach, Vice President
and Director, TMI
GPU Nuclear, Inc.
P.O. Box 480
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June 21, 1999

SUBJECT: THREE MILE ISLAND NUCLEAR STATION, UNIT 1 - ISSUANCE OF
AMENDMENT RE: MORE ACCURATELY REFLECT FACILITY DESIGN AND
INCORPORATE IMPROVEMENTS (TAC NO. M98522)

Dear Mr. Langenbach:

The Commission has issued the enclosed Amendment No. 212 to Facility Operating License No. DPR-50 for the Three Mile Island Nuclear Station, Unit No. 1, (TMI-1) in response to your application dated February 7, 1997, as supplemented October 24, 1998.

The amendment incorporates changes to more accurately reflect the current plant design, adopts changes in surveillance requirements consistent with the Standard Technical Specifications, identifies changes to plant systems and revisions to Technical Specifications system descriptions not involving Limiting Conditions for Operations, and makes editorial or typographical corrections.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Ronald B. Eaton, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-289

Enclosures: 1. Amendment No. 212 to DPR-50
2. Safety Evaluation

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*See previous concurrence

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

June 21, 1999

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and Director, TMI
GPU Nuclear, Inc.
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A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Ronald B. Eaton, Sr.", written over a horizontal line.

Ronald B. Eaton, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-289

Enclosures: 1. Amendment No. 212 to DPR-50
2. Safety Evaluation

cc w/encls: See next page

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Three Mile Island Nuclear Station, Unit No. 1

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UNITED STATES
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METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER & LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

GPU NUCLEAR, INC.

DOCKET NO. 50-289

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 212
License No. DPR-50

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear, Inc. et al., (the licensee), dated February 7, 1997, as supplemented October 24, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-50 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A, as revised through Amendment No. 212 , are hereby incorporated in the license. GPU Nuclear, Inc. shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



S. Singh Bajwa, Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: June 21, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 212

FACILITY OPERATING LICENSE NO. DPR-50

DOCKET NO. 50-289

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change.

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V

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4-5a

4-6

4-38

4-38a

4-39

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3.7 UNIT ELECTRIC POWER SYSTEM

Applicability

Applies to the availability of electrical power for operation of the unit auxiliaries.

Objective

To define those conditions of electrical power availability necessary to ensure:

- a. Safe unit operation
- b. Continuous availability of engineered safeguards

Specification

3.7.1 The reactor shall not be made critical unless all of the following requirements are satisfied:

- a. All engineered safeguards buses, engineered safeguards switchgear, and engineered safeguards load shedding systems are operable.
- b. One 7200 volt bus is energized.
- c. Two 230 kv lines are in service.
- d. One 230 kv bus is in services.
- e. Engineered safeguards diesel generators are operable and at least 25,000 gallons of fuel oil are available in the storage tank.
- f. Station batteries are charged and in service. Two battery chargers per battery are in service.

3.7.2 The reactor shall not remain critical unless all of the following requirements are satisfied:

- a. Two 230 kv lines are in service and capable of carrying auxiliary power to Unit 1, except as specified in Specification 3.7.2e below.
- b. Both 230/4.16 kv unit auxiliary transformers shall be in operation except that within a period not to exceed eight hours in duration from and after the time one Unit 1 auxiliary transformer is made or found inoperable, two diesel generators shall be operable, and one of the operable diesel generators will be started and run continuously until both unit auxiliary transformers are in operation. This mode of operation may continue for a period not exceeding 30 days.
- c. Both diesel generators shall be operable except that from the date that one of the diesel generators is made or found to be inoperable

for any reason, reactor operation is permissible for the succeeding seven days provided that the redundant diesel generator is:

1. verified to be operable immediately;
2. within 24 hours either:
 - a. determine the redundant diesel generator is not inoperable due to a common mode failure or
 - b. test redundant diesel generator in accordance with surveillance requirement 4.6.1.a.

In the event two diesel generators are inoperable, the unit shall be placed in hot shutdown in 12 hours. If one diesel is not operable within an additional 24 hour period the plant shall be placed in cold shutdown within an additional 24 hours thereafter.

With one diesel generator inoperable, in addition to the above, verify that: All required systems, subsystems, trains, components and devices that depend on the remaining OPERABLE diesel generator as a source of emergency power are also OPERABLE or follow specifications 3.0.1.

- d. If one Unit Auxiliary Transformer is inoperable and a diesel generator becomes inoperable, the unit will be placed in hot shutdown within 12 hours. If one of the above sources of power is not made operable within an additional 24 hours the unit shall be placed in cold shutdown within an additional 24 hours thereafter.
- e. If Unit 1 is separated from the system while carrying its own auxiliaries, or if only one 230 kv line is in service, continued reactor operation is permissible provided one emergency diesel generator shall be started and run continuously until two transmission lines are restored.
- f. The engineered safeguards electrical bus, switchgear, load shedding, and automatic diesel start systems shall be operable except as provided in Specification 3.7.2c above and as required for testing.
- g. One station battery may be removed from service for not more than eight hours.

TABLE 4.1-1 (Continued)

<u>CHANNEL DESCRIPTION</u>	<u>CHECK</u>	<u>TEST</u>	<u>CALIBRATE</u>	<u>REMARKS</u>
27. Makeup Tank Level Channels	D(1)	NA	F	(1) When Makeup and Purification System is in operation.
28. Radiation Monitoring Systems*				
a. RM-G6 (FH Bridge #1 Aux)	W(1)(2)	M(2)	Q(2)	(1) Using the installed check source when background is less than twice the expected increase in cpm which would result from the check source alone. Background readings greater than this value are sufficient in themselves to show that the monitor is functioning. (2) RM-G6 and RM-G7 operability requirements are given in T.S. 3.8.1. Surveillances are required to be current only when handling irradiated fuel. (3) RM-G9 operability requirements are given in T.S. 3.8.1. (4) RM-A2 operability requirements are given in T.S. 3.1.6.8.
b. RM-G7 (FH Bridge #2 Main)	W(1)(2)	M(2)	Q(2)	
c. RM-G9 (FH Bridge-FH Bldg)	W(1)(3)	M(3)	E(3)	
d. RM-A2P (RB Atmosphere particulate)	W(1)(4)	M(4)	E(4)	
e. RM-A2I (RB Atmosphere iodine)	W(1)(4)	M(4)	Q(4)	
f. RM-A2G (RB Atmosphere gas)	W(1)(4)	M(4)	E(4)	
29. High and Low Pressure Injection Systems: Flow Channels	N/A	N/A	F	

* Includes only monitors indicated under this item. Other T.S. required radiation monitors are included in specifications 3.5.5.2, 4.1.3, Table 3-5.1 item C.3.f, and Table 4.1-1 item 19e.

TABLE 4.1-1 (Continued)

	<u>CHANNEL DESCRIPTION</u>	<u>CHECK</u>	<u>TEST</u>	<u>CALIBRATE</u>	<u>REMARKS</u>
30.	Borated Water Storage Tank Level Indicator	W	NA	F	
31.	Boric Acid Mix Tank				
	a. Level Channel	NA	NA	F	
	b. Temperature Channel	M	NA	F	
32.	Reclaimed Boric Acid Storage Tank				
	a. Level Channel	NA	NA	F	
	b. Temperature Channel	M	NA	F	
33.	Containment Temperature	NA	NA	F	
34.	Incore Neutron Detectors	M(1)	NA	NA	(1) Check functioning; including functioning of computer readout or recorder readout when reactor power is greater than 15%.
35.	Emergency Plant Radiation Instruments	M(1)	NA	F	(1) Battery check.
36.	(DELETED)				
37.	Reactor Building Sump Level	NA	NA	R	

4.4.4 Hydrogen Recombiner System

Applicability

Applies to the testing of the hydrogen recombiner and associated controls.

Objective

To verify that the hydrogen recombiner and associated controls are operable.

4.4.4.1 Specification

- a. Perform a system functional test for each hydrogen recombiner each refueling interval as follows:
 - (1) Verify that the minimum heater sheath temperature increases to $\geq 700^{\circ}\text{F}$ in ≤ 90 minutes.
 - (2) After reaching 700°F , increase the power to maximum for approximately 2 minutes and verify the power to be $\geq 60\text{kW}$.
- b. Visually examine the hydrogen recombiner enclosure and verify there is no evidence of abnormal conditions each refueling interval.
- c. Perform a resistance to ground test for each heater phase each refueling interval and verify that the resistance to ground for any heater is $\geq 10,000$ ohms.

Bases

The surveillance program described above provides high assurance that the hydrogen recombiner system will be available to perform its post-LOCA function of maintaining the containment hydrogen concentration below 4.1 volume percent. This system is not credited to mitigate any accident analyzed in Chapter 14 of the TMI-1 FSAR. The frequency of the surveillance of the hydrogen recombiner system is based on the safety significance of the system. TMI-1 FSAR Section 6.5.3.1 indicates that the hydrogen recombiner system is not required until 9.0 days following a LOCA. This is adequate time to place a hydrogen recombiner in service.

4.5 EMERGENCY LOADING SEQUENCE AND POWER TRANSFER, EMERGENCY CORE COOLING SYSTEM & REACTOR BUILDING COOLING SYSTEM PERIODIC TESTING

4.5.1 Emergency Loading Sequence

Applicability: Applies to periodic testing requirements for safety actuation systems.

Objective: To verify that the emergency loading sequence and automatic power transfer is operable.

Specifications:

4.5.1.1 Sequence and Power Transfer Test

- a. During each refueling interval, a test shall be conducted to demonstrate that the emergency loading sequence and power transfer is operable.
- b. The test will be considered satisfactory if the following pumps and fans have been successfully started and the following valves have completed their travel on preferred power and transferred to the emergency power.

- M. U. Pump
- D. H. Pump and D. H. Injection Valves and D. H. Supply Valves
- R. B. Cooling Pump
- R. B. Ventilators
- D. H. Closed Cycle Cooling Pump
- N. S. Closed Cycle Cooling Pump
- D. H. River Cooling Pump
- N. S. River Cooling Pump
- D. H. and N. S. Pump Area Cooling Fan
- Screen House Area Cooling Fan
- Spray Pump. (Initiated in coincidence with a 2 out of 3 R. B. 30 psig Pressure Test Signal.)
- Motor Driven Emergency Feedwater Pump

- c. Following successful transfer to the emergency diesel, the diesel generator breaker will be opened to simulate trip of the generator then re-closed to verify block load on the reclosure.

4.5.1.2 Sequence Test

- a. At intervals not to exceed 3 months, a test shall be conducted to demonstrate that the emergency loading sequence is operable, this test shall be performed on either preferred power or emergency power.
- b. The test will be considered satisfactory if the pumps and fans listed in 4.5.1.1b have been successfully started and the valves listed in 4.5.1.1b have completed their travel.

4.5.3

REACTOR BUILDING COOLING AND ISOLATION SYSTEM

Applicability

Applies to testing of the reactor building cooling and isolation systems.

Objective

To verify that the reactor building cooling systems are operable.

Specification

4.5.3.1

System Tests

a. Reactor Building Spray System

1. At each refueling interval, a reactor building 30 psi high pressure test signal will start the spray pump. Except for the spray pump suction valves, all engineered safeguards spray valves will be closed.

Water will be circulated from the borated water storage tank through the reactor building spray pumps and returned through the test line to the borated water storage tank.

The operation of the spray valves will be verified during the component test of the R. B. cooling and isolation system.

The test will be considered satisfactory if the spray pumps have been successfully started.

2. Compressed air will be introduced into the spray headers to verify each spray nozzle is unobstructed at least every ten years.

b. Reactor Building Cooling and Isolation Systems

1. During each refueling period, a system test shall be conducted to demonstrate proper operation of the system. A test signal will actuate the Reactor Building Emergency Cooling System valves to demonstrate operability of the coolers.
2. The test will be considered satisfactory if the valves have completed their expected travel.

5.5 AIR INTAKE TUNNEL FIRE PROTECTION SYSTEMS

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6.3 UNIT STAFF QUALIFICATIONS

- 6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1 of 1978 for comparable positions unless otherwise noted in the Technical Specifications. Licensed operators shall also meet the requirements of 10 CFR Part 55. Individuals who do not meet ANSI/ANS 3.1 of 1978, Section 4.5, are not considered technicians or maintenance personnel for purposes of determining qualifications but are permitted to perform work for which qualification has been demonstrated.
- 6.3.2 The management position responsible for radiological controls shall meet or exceed the qualifications of Regulatory Guide 1.8 of 1977. Each radiological controls technician/supervisor shall meet or exceed the qualifications of ANSI-N 18.1-1971, paragraph 4.5.2/4.3.2, or be formally qualified through an NRC approved TMI-1 Radiation Controls training program. All radiological controls technicians will be qualified through training and examination in each area or specific task related to their radiological controls functions prior to their performance of those tasks.
- 6.3.3 The Shift Technical Advisors shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in unit design, response and analysis of transients and accidents.

6.4 TRAINING

- 6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the plant training manager and shall meet or exceed the requirements and recommendations of Regulatory Guide 1.8 of 1977. Licensed operator training shall also meet the requirements of 10 CFR Part 55.
- 6.4.2 A training program for the Fire Brigade shall be maintained and shall meet or exceed the requirements of Section 27 of the NFPA Code - 1976.

6.5 REVIEW AND AUDIT

6.5.1 TECHNICAL REVIEW AND CONTROL

The Vice President of each division within GPU Nuclear, Inc. shall be responsible for ensuring the preparation, review, and approval of documents required by the activities described in 6.5.1.1 through 6.5.1.5 within his functional area of responsibility as assigned in the GPU Nuclear, Inc. Review and Approval Matrix. Implementing approvals shall be performed at the cognizant manager level or above.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 212 TO FACILITY OPERATING LICENSE NO. DPR-50

METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER & LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

GPU NUCLEAR, INC.

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-289

1.0 INTRODUCTION

By letter dated February 7, 1997, as supplemented October 24, 1998, GPU Nuclear, Inc. (the licensee) submitted a request for changes to the Three Mile Island Nuclear Station, Unit 1 (TMI-1) Technical Specifications (TSs). The requested changes would incorporate changes to more accurately reflect current plant design, adopt changes in surveillance requirements consistent with the Standard Technical Specifications (STS), identify changes to plant systems and revisions to Technical Specifications (TS) system descriptions not involving Limiting Conditions for Operations (LCOs), and make editorial or typographical corrections. The October 24, 1998, letter provided additional clarifying information which did not expand the scope of the Federal Register notice. The letter also withdrew the amendment request relating to the hydrogen recombiners.

2.0 EVALUATION

The licensee proposed the following changes to the TMI-1 TSs.

2.1 TS 3.7.2.b

TS 3.7.2.b currently requires, in part, that in lieu of running a diesel generator, a 4160-volt tie from a Unit 2 transformer shall be placed in service, thus supplying the second of two feeds to the engineered safeguard buses causing no degradation of the system and permitting continued operation indefinitely should one of the auxiliary transformers either be made or found inoperable. The licensee has proposed to delete reference to the tie from the TMI-2 transformer from TS 3.7.2.b. The licensee states that the modification to provide this capability to tie Unit 2 was abandoned prior to its completion, and the Unit 2 transformers have been removed from the site. The staff finds the proposed change to be acceptable since no tie from the Unit 2 transformer was ever installed and the Unit 2 transformer has been removed from the site. Additionally, all other TS related to electrical power supplies remains the same.

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2.2 TS 3.7.2.d

TS 3.7.2.d currently requires, in part, that if one of the TMI-1 Unit Auxiliary transformers is inoperable and a 4160-volt tie from the Unit 2 transformer cannot be placed in service and a diesel generator becomes inoperable, the unit will be placed in hot shutdown within 12 hours. The licensee has proposed to delete reference to the 4160-volt tie from the Unit 2 transformer. The staff finds the proposed change to be acceptable since no tie from the Unit 2 transformer was ever installed, the requirements related to the diesel generators remains the same and the Unit 2 transformer has been removed from the site.

2.3 Surveillance Requirement 4.5.1.1.b

Surveillance Requirement (SR) 4.5.1.1.a requires demonstration of the "as designed" operation of the diesel generators during the loss of the offsite power source. The "as designed" operations include all actions resulting from the loss of offsite power source, including the shedding of non-essential loads and energizing of emergency buses and respective loads from the diesel generator. SR 4.5.1.1.b currently requires that the test will be satisfactory if the pumps and fans have been successfully started and the valves have completed their travel on preferred power and transferred to emergency power as evidence by the control board component operating lights, and a second means of verification, such as: the station computer, verification of pressure/flow, or control board indicating lights initiated by separate limit switch contacts. The licensee has proposed to delete the requirement to verify component operability by control board operating lights and a second means, from the SR.

The licensee states that the SR is demonstrated through the performance of Surveillance Procedure 1303-11.10, entitled "ES System Emergency Sequence and Power Transfer Test." Specific steps in this procedure direct operators to test, obtain evidence, and document that the requirements were met. The staff finds the above change to be acceptable because the specific wording to evidence the successful starting of pumps, fans, and completion of valve travel need not be included in the TS as they are included in plant specific procedures.

2.4 SR 4.5.1.2.b

The licensee has proposed to revise SR 4.5.1.2.b to delete the specific wording to evidence the equipment operability in the same manner as 4.5.1.1.b above. The staff finds the proposed change to be acceptable for the same reasons, as the plant specific procedures adequately demonstrate the performance of the SR and, therefore, it does not need to be included in the TS.

2.5 SR 4.5.3.1.a.1

SR 4.5.3.1.a.1 currently requires, in part, that at each refueling interval and simultaneously with the test of the emergency loading sequence, a Reactor Building (RB) 30 psi high-pressure test signal will start the spray pump. Except for the spray pump suction valves, all engineered safeguards spray valves will be closed. The test will be considered satisfactory if the spray pumps have been successfully started as evidenced by the control board component operating

lights, and either the station computer or pressure/flow indication. The licensee has proposed to delete both the requirements to perform the tests simultaneously and the need to evidence the successful starting of the spray pump and the completion of expected travel of affected valves.

The licensee states that the requirement to test the RB spray pump simultaneously with the test of the emergency loading sequence is considered unnecessary and overly prescriptive. The staff finds there is no technical requirement to perform both tests simultaneously, and agrees that the requirement could be considered overly prescriptive, therefore, the proposed revised wording to eliminate the requirement for simultaneous performance of the tests is acceptable.

2.6 SR 4.5.3.1.a.1 and SR 4.5.3.1.b.2

SRs 4.5.3.1.a.1 and 4.5.3.1.b.1 require a system test during the refueling period to demonstrate proper operation of the Reactor Building Emergency Cooling Systems. Both SRs currently state that the test will be considered satisfactory if the spray pump (SR 4.5.3.1.a.1) and the valves have completed their expected travel (SR 4.5.3.1.b.2) as evidenced by the control board component operating lights and a second means of verification, such as: the station computer, verification of pressure/flow, or control board indicating lights initiated by separate limit switch contacts. The licensee has proposed to delete the requirement to verify component operability by control board operating lights and a second means, from both SRs. The licensee provided a safety analysis for 4.5.3.1.a.1 but did not include a safety analysis for the identical request for SR 4.5.3.1.b.2. Both SRs were included in the request and were appropriately noticed, therefore, the staff did evaluate the request. The licensee's safety analysis for 4.5.3.1.a.1 equally applies to 4.5.3.1.b.2. The staff finds the request acceptable based on the fact that both SRs are demonstrated by performance of procedures with specific steps that obtain evidence and document that the requirements were met. Therefore, the specific wording and additional verification does not need to be in the TSs.

2.7 Strong Motion Accelerometer

The licensee has proposed to delete Item 36, Strong Motion Accelerometer, from Table 4.1-1 and its associated SR. The request is based on Generic Letter (GL) 95-10, "Relocation of Selected Technical Specifications Requirements Related to Instrumentation," incorporation of the battery check and calibration requirements into the plant's preventive program, and the absence of the requirements in the STSs. The GL included the seismic monitoring instrumentation as one of the candidates that met the criteria in 10 CFR 50.36 for relocation in a plant specific procedure and deletion from TSs.

The staff has reviewed their proposal and the additional information provided and determined that the deletion of the TS based on the inclusion of the plant-specific procedure 1302-3.2, "Strong Motion Accelerometer Battery and Calibration" in the preventive maintenance program is acceptable. This item does not meet the criteria for inclusion in TSs pursuant to 10 CFR 50.36. Additionally, though not required, the licensee indicated that it will implement the recommendations contained in the Electric Power Research Institute (EPRI) document EPRI-NP-6695, "Guidelines for Nuclear Plant Response to an Earthquake."

2.8 Section 5.5

The licensee requested deletion of the TS requirement for section 5.5 and relocation of it to the final safety analysis report (FSAR). The description of the equipment contained in section 5.5 will be relocated to FSAR Section 9.9.2. The staff has reviewed Section 9.9 of the FSAR and finds the description of the Air Intake Tunnel Fire Protection System complete with respect to the description in Section 5.5 of the TSs. The staff finds it acceptable to remove the descriptive information from the TSs based on it already being incorporated into the FSAR. According to the information submitted by the licensee, the current testing methodology and frequency are unaffected by the proposed change since the implementation of surveillance testing is required to be maintained by the fire protection program and accomplished under administrative procedure 1001J, "Technical Specification Surveillance Testing Program."

Amendment No. 146 to Facility Operating License No. DPR-50 for Three Mile Island, Unit 1 dated November 30, 1988, incorporated and satisfied the five elements of the guidance contained in GL 88-12, "Removal of Fire Protection Requirements from Technical Specifications." The NRC-approved changes relocated requirements for fire detection systems, fire suppressions systems, fire barriers, and fire brigade staffing requirements as recommended by GL 86-10, "Implementation of Fire Protection Requirements." At that time, the licensee did not relocate the requirements from TS section 5.5, "Air Intake and Tunnel Fire Protection Systems."

The staff reviewed the licensee's amendment request against the guidance provided in GLs 86-10, 88-12, and Amendment No. 146. On the basis of its review, the staff concludes that the proposed deletion of TS section 5.5 meets the guidance of GLs 86-10 and 88-12, and is consistent with the staff's approval of Amendment No. 146. The section to be eliminated does not meet the criteria for inclusion in the TSs pursuant to 10 CFR 50.36. The proposed change is therefore, acceptable.

2.9 Hydrogen Recombiner System

By letter dated October 24, 1998, the licensee withdrew the portion of the TS change request related to the hydrogen recombiners.

2.10 Editorial and Typographical Corrections

The licensee made four editorial/typographical corrections consisting of changes to: (1) Table 3-5.1 to Table 3.5-1 in the foot note on page 4-5a, (2) 9.8 days to 9.0 days on page 4-38, (3) Section 27 of the NFPA Code - 1976 to Section 600 of the NFPA Code, and (4) grater to greater on page 4.6. The staff has reviewed and agrees with these corrections.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (63 FR 14486). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

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Date: June 21, 1999

DATED: June 21, 1999

AMENDMENT NO. 212 TO FACILITY OPERATING LICENSE NO. DPR-50 THREE MILE ISLAND

~~Docket File~~

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