

Mr. James W. Langenbach, Vice President
and Director, TMI
GPU Nuclear, Inc.
P.O. Box 480
Middletown, PA 17057

September 23, 1999

SUBJECT: CORRECTION LETTER - AMENDMENT NO. 215 ISSUED ON AUGUST 24, 1999
RE: ENGINEERED SAFEGUARDS FEATURE (ESF) SYSTEM LEAKAGE LIMIT
(TAC NO. MA4665)

Dear Mr. Langenbach:

The Commission issued Amendment No. 215 to Facility Operating License No. DPR-50 for the Three Mile Island Nuclear Station, Unit 1 (TMI-1), on August 24, 1999, in response to your application dated February 2, 1999, as supplemented by letter July 29, 1999.

The amendment allows an increased ESF system leakage for the Cycle 13 operating cycle. In your letter dated September 1, 1999, you indicated that you made an error in your July 29, 1999, supplement to your original application and inadvertently used an earlier uncontrolled version of Technical Specification (TS) page 4-45 when adding a footnote to limit the change as applicable during Cycle 13 operation. We understand that you had intended that the page would otherwise be the same as included in your February 2, 1999, application. Your September 1, 1999, letter included a replacement page 4-45 to correct the error and requested that we issue an errata letter to reflect your intended amendment request. The staff has reviewed your request and agrees that the September 1, 1999, revised page 4-45 corrects the error contained in your July 29, 1999, supplement to your February 2, 1999, application. Enclosed is corrected page 4-45 for TS Section 4.5.4. The staff notes from conversations with your staff that you have initiated corrective actions to prevent errors of this type from occurring in the future.

Sincerely,

ORIGINAL SIGNED BY:

Timothy G. Colburn, Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-289

Enclosure: TS page 4-45

cc w/encl: See next page

DFOI //

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Three Mile Island Nuclear Station, Unit No. 1

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Applicability

Applies to those portions of the Decay Heat, Building Spray, and Make-Up Systems, which are required to contain post accident sump recirculation fluid, when these systems are required to be operable in accordance with Technical Specification 3.3.

Objective

To maintain a low leakage rate from the ESF systems in order to prevent significant off-site exposures and dose consequences.

Specification

4.5.4.1

The total maximum allowable leakage into the Auxiliary Building from the applicable portions of the Decay Heat, Building Spray and Make-Up System components as measured during refueling interval tests in Specification 4.5.4.2 shall not exceed 15 gallons per hour.*

4.5.4.2

Once each refueling interval the following tests of the applicable portions of the Decay Heat Removal, Building Spray and Make-Up Systems shall be conducted to determine leakage:

- a. The applicable portion of the Decay Heat Removal System that is outside containment shall be leak tested with the Decay Heat pump operating, except as specified in "b".
- b. Piping from the Reactor Building Sump to the Building Spray pump and Decay Heat Removal System pump suction isolation valves shall be pressure tested at no less than 55 psig.
- c. The applicable portion of the Building Spray system that is outside containment shall be leak tested with the Building Spray pumps operating and BS-V-1A/B closed, except as specified in "b" above.
- d. The applicable portion of the Make-Up system on the suction side of the Make-Up pumps shall be leak tested with a Decay Heat pump operating and DH-V-7A/B open.
- e. The applicable portion of the Make-Up system from the Make-Up pumps to the containment boundary valves (MU-V-16A/D, 18, and 20) shall be leak tested with a Make-Up pump operating.
- f. Visual inspection shall be made for leakage from components of these systems. Leakage shall be measured by collection and weighing or by another equivalent method.

*NOTE: This leak rate limit is only applicable for the Cycle 13 operating cycle.

Bases

The leakage rate limit of 15 gph (measured in standard room temperature gallons) for the accident recirculation portions of the Decay Heat Removal (DHR), Building Spray (BS), and Make-Up (MU) systems is based on ensuring that potential leakage after a loss-of-coolant accident will not result in off-site dose consequences in excess of those calculated to comply with the 10 CFR 100 limits (Reference 1 and 2). The test methods prescribed in 4.5.4.2 above for the applicable portions of the DH, BS and MU systems ensure that the testing results account for the highest pressure within that system during the sump recirculation phase of a design basis accident.

References

- (1) UFSAR, Section 6.4.4 - "Design Basis Leakage;" and, Table 6.4-3 - "Leakage Quantities to the Auxiliary Building"
- (2) UFSAR, Section 14.2.2.5(d) - "Effects of Engineered Safeguards Leakage During Maximum Hypothetical Accident"