

August 12, 1997

Mr. James W. Langenbach, Vice President
and Director - TMI-1
GPU Nuclear Corporation
P.O. Box 480
Middletown, PA 17057

SUBJECT: THREE MILE ISLAND - ISSUANCE OF AMENDMENT RE: INCORPORATING
TOPICAL REPORTS TR-078-A, REVISION 0, TR-087-A, REVISION 0,
TR-091-A, REVISION 0, AND TR-092P-A, REVISION 0, INTO
TECHNICAL SPECIFICATIONS (TAC NO. M98733)

Dear Mr. Langenbach:

The Commission has issued the enclosed Amendment No. 202 to Facility Operating
License No. DPR-50 for the Three Mile Island Nuclear Station, Unit No. 1,
(TMI-1) in response to your application dated May 8, 1997, as supplemented
June 10, and July 25, 1997.

The amendment incorporates into the TMI-1 Technical Specifications NRC-
approved analytical methods used to determine TMI-1 core operating limits.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance
will be included in the Commission's biweekly Federal Register notice.

This concludes our effort on this issue and we are, therefore, closing out TAC
No. M98733.

Sincerely,

Original signed by:

Bart C. Buckley, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-289

Enclosures: 1. Amendment No. 202 to DPR-50
2. Safety Evaluation

cc w/encls: See next page

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Three Mile Island Nuclear Station, Unit No. 1

cc:

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DATED: August 12, 1997

AMENDMENT NO. 202 TO FACILITY OPERATING LICENSE NO. DPR-50 THREE MILE ISLAND

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER & LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

GPU NUCLEAR CORPORATION

DOCKET NO. 50-289

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 202
License No. DPR-50

1. The Nuclear Regulatory Commission (the Commission or NRC) has found that:
 - A. The application for amendment by GPU Nuclear Corporation, et al. (the licensee) dated May 8, 1997, as supplemented June 10, and July 25, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

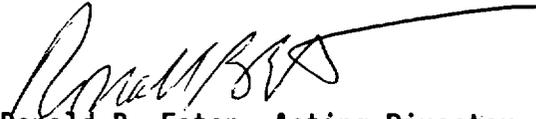
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-50 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 202, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Ronald B. Eaton, Acting Director
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: August 12, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 202

FACILITY OPERATING LICENSE NO. DPR-50

DOCKET NO. 50-289

Replace the following page of the Appendix A, Technical Specifications, with the attached page. The revised page is identified by Amendment number and contains vertical lines indicating the areas of change.

Remove

6-19

Insert

6-19

6.9.5 **CORE OPERATING LIMITS REPORT**

6.9.5.1 The core operating limits addressed by the individual Technical Specifications shall be established and documented in the CORE OPERATING LIMITS REPORT prior to each reload cycle or prior to any remaining part of a reload cycle.

6.9.5.2 The analytical methods used to determine the core operating limits addressed by the individual Technical Specifications shall be those previously reviewed and approved by the NRC for use at TMI-1, specifically:

- (1) BAW-10179 P-A, "Safety and Methodology for Acceptable Cycle Reload Analyses." The current revision level shall be specified in the COLR.
- (2) TR-078-A, "TMI-1 Transient Analyses Using the RETRAN Computer Code", Revision 0. NRC SER dated 2/10/97.
- (3) TR-087-A, "TMI-1 Core Thermal-Hydraulic Methodology Using the VIPRE-01 Computer Code", Revision 0. NRC SER dated 12/19/96.
- (4) TR-091-A, "Steady State Reactor Physics Methodology for TMI-1", Revision 0. NRC SER dated 2/21/96.
- (5) TR-092P-A, "TMI-1 Reload Design and Setpoint Methodology", Revision 0. NRC SER dated 4/22/97.

6.9.5.3 The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient/accident analysis limits) of the safety analysis are met.

6.9.5.4 The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance for each reload cycle to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 202 TO FACILITY OPERATING LICENSE NO. DPR-50
METROPOLITAN EDISON COMPANY
JERSEY CENTRAL POWER & LIGHT COMPANY
PENNSYLVANIA ELECTRIC COMPANY
GPU NUCLEAR CORPORATION
THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1
DOCKET NO. 50-289

1.0 INTRODUCTION

By submittal dated May 8, 1997, as supplemented June 10, and July 25, 1997, GPU Nuclear Corporation (the licensee) requested an amendment to the Three Mile Island Nuclear Generating Station, Unit 1 (TMI-1) Technical Specifications (TS). The amendment incorporates into the TMI-1 TS additional NRC-approved analytical methods used to determine TMI-1 core operating limits.

The June 10, and July 25, 1997, letter provided clarifying information that did not change the scope of the May 8, 1997, submittal and the proposed no significant hazards consideration.

2.0 EVALUATION

The NRC staff has previously evaluated and approved the use of the analytical methods contained in the following Topical Reports to determine core operating limits at TMI-1:

| <u>Topical Reports</u> | <u>NRC SER Issue Date</u> |
|---|---------------------------|
| TR-078-A, Revision 0, "TMI-1 Transient Analyses Using the RETRAN Computer Code" | February 10, 1997* |
| TR-087-A, Revision 0, "TMI-1 Core Thermal-Hydraulic Methodology Using the VIPRE-01 Computer Code" | December 19, 1996* |
| TR-091-A, Revision 0, "Steady State Reactor Physics Methodology for TMI-1" | February 21, 1996 |
| TR-092P-A, Revision 0, "TMI-1 Reload Design and Setpoint Methodology" | April 22, 1997* |

* Approval in these safety evaluations is based on the current licensing basis for TMI-1.

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The proposed amendment incorporates the above-cited topical reports into TS 6.9.5 that sets forth the requirements for documentation of core operating limits in the Core Operating Limits Report, which is submitted to the NRC upon issuance for each reload. This change is acceptable since the analytical methodologies delineated in each of these reports have been approved by the NRC for use in TMI-1 applications. Implementation of these analytical methodologies for TMI-1 is governed by the limitations and requirements specified in the individual safety evaluations approving each of the topical reports. The use of these NRC-approved methodologies will ensure that values for cycle-specific parameters are determined such that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, emergency core cooling system limits, nuclear limits such as shutdown margin, and transient/accident analysis limits) of the safety analysis are met.

3.0 SUMMARY CONCLUSION

Based on the NRC staff's evaluation of the licensee's submittals, the staff finds the proposed amendment request acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Bart Buckley

Date: August 12, 1997