

March 9, 1987

DPR
o/c

Docket No. 50-289

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Dear Mr. Hukill:

The Commission has issued the enclosed Amendment No.125 to Facility Operating License No. DPR-50 for the Three Mile Island Nuclear Station, Unit No. 1 (TMI-1). This amendment consists of changes to the Technical Specifications (TSs) in response to your letter dated November 3, 1986, as supplemented November 19, 1986.

This amendment revises TS 3.4.1.2, and associated bases, to allow plant operation with a minimum of two operable Main Steam Safety Valves per steam generator when the reactor has been subcritical for at least one hour and reactor coolant temperatures range from 250°F to Hot Shutdown conditions (i.e., greater than 525°F and reactor subcritical).

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Thierry M. Ross, Project Manager
PWR Project Directorate #6
Division of PWR Licensing-B

Enclosures:

1. Amendment No.125 to DPR-50
2. Safety Evaluation

cc w/enclosures:
See next page

PBD-6 RIngram 2/26/87	<i>501</i> PBD-6 JThoma:eh 2/26/87	<i>Raw</i> PBD-6 TRoss 2/24/87	<i>Raw</i> PBD-6 RWeller 2/26/87	<i>[Signature]</i> PBD-6 JStolz 2/26/87	<i>No legal objection.</i> OGC <i>[Signature]</i> S H Lewis 3/15/87
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Atomic Safety & Licensing Appeal
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WASHINGTON, D. C. 20555

METROPOLITAN EDISON COMPANY

NEW JERSEY CENTRAL POWER AND LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

GPU NUCLEAR CORPORATION

DOCKET NO. 50-289

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 125
License No. DPP-50

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation, et al. (the licensees) dated November 3, 1986, as supplemented November 19, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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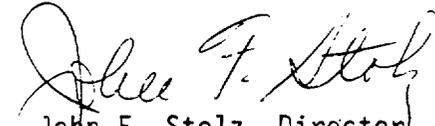
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-50 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 125, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Director
PWR Project Directorate #6
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 9, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 125

FACILITY OPERATING LICENSE NO. DPR-50

DOCKET NO. 50-289

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and certain vertical lines indicating the area of change.

Remove

3-26
3-26a
3-26b
3-26c

Insert

3-26
3-26a
3-26b
3-26c

- 3.4.1.2.1 With the Reactor from 250°F to HOT SHUTDOWN and subcritical for at least one (1) hour, two (2) Main Steam Safety Valves per Steam Generator shall be OPERABLE. With less than two (2) Main Steam Safety Valves per Steam Generator OPERABLE, restore at least two (2) MSS Valves to OPERABLE status for each Steam Generator within 6 hours or be in COLD SHUTDOWN within the following 30 hours.
- 3.4.1.2.2 When the Reactor is above HOT SHUTDOWN, all eighteen (18) Main Steam Safety Valves shall be OPERABLE or, if any are not OPERABLE, the maximum overpower trip setpoint (see Table 2.3-1) shall be reset as follows:

<u>Maximum Number of Safety Valves Disabled on Any Steam Generator</u>	<u>Maximum Overpower Trip Setpoint (% of Rated Power)</u>
1	92.4
2	79.4
3	66.3

With more than three (3) Main Steam Safety Valves INOPERABLE, restore at least fifteen (15) Main Steam Safety Valves to OPERABLE status within 4 hours or be in at least HOT SHUTDOWN within the next 6 hours.

- 3.4.2 Reactor Coolant System temperature 250°F or less.
- 3.4.2.1 With Reactor Coolant temperature 250°F or less, at least two of the following means for maintaining decay heat removal capability shall be OPERABLE and at least one shall be in operation except as allowed by Specifications 3.4.2.2, 3.4.2.3 and 3.4.2.4.
- a. Decay Heat Removal String "A".
 - b. Decay Heat Removal String "B".
 - c. Reactor Coolant Loop "A", its associated OTSG, and its associated emergency feedwater flowpath.
 - d. Reactor Coolant Loop "B", its associated OTSG, and its associated emergency feedwater flowpath.
- 3.4.2.2 Operation of the means for decay heat removal may be suspended provided the core outlet temperature is maintained below saturation temperature.
- 3.4.2.3 The number of means for decay heat removal required to be operable per 3.4.2.1 may be reduced to one provided that one of the following conditions is satisfied:
- a. The Reactor is in a Refueling Shutdown condition with the Fuel Transfer Canal water level greater than 23 feet above the reactor vessel flange.

- b. Reactor coolant temperature is less than 140°F with BWST level greater than 44 feet and an associated flow path through the RCS OPERABLE such that core outlet temperature can be maintained subcooled for at least 7 days.
- c. Equipment Maintenance on one of the means for decay heat removal specified by 3.4.2.1 is required and the equipment outage does not exceed 7 days.

3.4.2.4 Specification 3.4.2.1 does not apply when either of the following conditions exist:

- a. Decay heat generation is less than 188 KW with the RCS full.
- b. Decay heat generation is less than 100 KW with the RCS drained down for maintenance.

3.4.2.5 With less than the above required loops OPERABLE, immediately initiate corrective action to return the required loops to OPERABLE status as soon as possible.

Bases

A reactor shutdown following power operation requires removal of core decay heat. Normal decay heat removal is by the steam generators with the steam dump to the condenser when RCS temperature is above 250°F and by the decay heat removal system below 250°F. Core decay heat can be continuously dissipated up to 15 percent of full power via the steam bypass to the condenser as feedwater in the steam generator is converted to steam by heat absorption. Normally, the capability to return feedwater flow to the steam generators is provided by the main feedwater system.

The main steam safety valves will be able to relieve to atmosphere the total steam flow if necessary. During hot shutdown or below, only a minimum number of Main Steam Safety Valves need to be operable as stated in Technical Specification 3.4.1.2.1. This is to provide Steam Generator overpressure protection during hot shutdown when hot functional testing is being performed. The minimum number of valves required to be operable allows margin for testing without jeopardizing plant safety. Plant specific analysis shows that one Main Steam Safety Valve is sufficient to relieve reactor coolant pump heat and stored energy when the reactor is subcritical by 1% ΔK/K for at least one hour. One MSS valve is required by the 1968 ASME Boiler and Pressure Vessel Code Section III, Article 9, Paragraph N-910.4 to relieve steam pressure. However, two MSSVs per Steam Generator will provide conservative redundancy. During power operations, if Main Steam Safety Valves are inoperable, the power level must be reduced, as stated in Technical Specification 3.4.1.2.2 such that the remaining safety valves can prevent overpressure on a turbine trip.

In the unlikely event of complete loss of off-site electrical power to the station, decay heat removal is by either the steam-driven emergency feedwater pump, or two half-sized motor-driven pumps. Steam discharge is to the atmosphere via the Main Steam Safety Valves and controlled atmospheric relief valves, and in the case of the turbine driven pump, from the turbine exhaust.(1)

Both motor-driven EFW pumps, or the steam-driven EFW pump are required initially to remove decay heat with one EFW pump eventually sufficing. The minimum amount of water in the condensate storage tanks, contained in Technical Specification 3.4.1.1., will allow cooldown to 250°F with steam being discharged to the atmosphere. After cooling to 250°F, the decay heat removal system is used to achieve further cooling.

When the RCS is below 250°F, a single DHR string, or single OTSG and its associated emergency feedwater flowpath is sufficient to provide removal of decay heat at all times following the cooldown to 250°F. The requirement to maintain two OPERABLE means of decay heat removal ensures that a single failure does not result in a complete loss of decay heat removal capability. The requirement to keep a system in operation as necessary to maintain the system subcooled at the core outlet provides the guidance to ensure that steam conditions which could inhibit core cooling do not occur.

Limited reduction in redundancy is allowed for preventive or corrective maintenance on the primary means for decay heat removal to ensure that maintenance necessary to assure the continued reliability of the systems may be accomplished.

As decay heat loads are reduced through decay time or fuel off loading, alternate flow paths will provide adequate cooling for a time sufficient to take compensatory action if the normal means of heat removal is lost.

With the reactor vessel head removed and 23 feet of water above the reactor vessel flange, a large heat sink is available for core cooling. The BWST with level at 44 feet provides an equivalent reservoir available as a heat sink. Operability of the BWST is to be determined using calculations based on actual plant data or through plant testing at the time the system is to be declared operable. At such times that either of these means is determined to be operable, removal of the redundant or diverse cooling system is permitted.

Following extensive outages or major core off loading, the decay heat generation being removed from the Reactor Vessel is so low that ambient losses are sufficient to maintain core cooling and no other means of heat removal is required. The system is passive and requires no redundant or diverse backup system. Decay heat generation is calculated in accordance with ANSI 5.1-1979 to determine when this situation exists.

An unlimited emergency feedwater supply is available from the river via either of the two motor-driven reactor building emergency cooling water pumps for an indefinite period of time.

The requirements of Technical Specification 3.4.1.1 assure that before the reactor is heated to above 250°F, adequate auxiliary feedwater capability is available. One turbine driven pump full capacity (920 gpm) and the two half-capacity motor-driven pumps (460 gpm each) are specified. However, only one half-capacity motor-driven pump is necessary to supply auxiliary feedwater flow to the steam generators in the onset of a small break loss-of-coolant accident.

REFERENCES

(1) FSAR Section 10.2.1.3



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 125 TO FACILITY OPERATING LICENSE NO. DPR-50

METROPOLITAN EDISON COMPANY
BERSEY CENTRAL POWER AND LIGHT COMPANY
PENNSYLVANIA ELECTRIC COMPANY
GPU NUCLEAR CORPORATION

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-289

INTRODUCTION

By letter dated November 3, 1986, as supplemented November 19, 1986, GPU Nuclear Corporation (GPU or the licensee) requested an amendment of the Technical Specifications (TSs) appended to Facility Operating License No. DPR-50 for the Three Mile Island Nuclear Station, Unit No. 1 (TMI-1). The proposed amendment would revise TS 3.4.1.2 to permit plant operation with only two Main Steam Safety Valves (MSSVs) per steam generator operable when the Reactor Coolant System (RCS) temperature is between 250°F and that corresponding to hot shutdown conditions (i.e., PCS is greater than 525°F and reactor subcritical). Additionally, the reactor must have been subcritical for at least one hour before the new MSSV operability requirements would apply. This change will allow for higher plant pressure and temperature conditions in order to conduct MSSV testing while the reactor is subcritical. When the reactor is above hot shutdown conditions, or has been subcritical for less than one hour, the current TS operability requirements apply.

The MSSVs provide secondary system overpressure protection during all modes of plant operation. Currently, TS 3.4.1.2 requires all 18 MSSVs to be operable for plant operation with RCS temperature above 250°F. If any of the MSSVs are not operable, the maximum overpower trip setpoint is lowered per the requirements of the TS 3.4.1.2. The licensee has asserted that the current TS does not provide conditions for MSSV testing after valve maintenance during the cold shutdown or refueling modes since main steam pressure of approximately 900 psig (saturation temperature 534°F) is needed for testing of the MSSVs.

EVALUATION

The licensee, in a letter dated November 19, 1986, provided the results of an analysis which demonstrates that one MSSV is sufficient to relieve the amount of heat added by the reactor coolant pumps and the decay heat generated from the core one hour following a reactor trip. The licensee's analysis assumes: (1) steam conditions are dry saturated, (2) atmospheric dump valves remain closed, (3) the reactor has been subcritical for at least one hour, and (4) a decay heat value of 1.2 times the ANS standard (ANS 5.1, 1979).

Based on the data for the MSSV relieving capacity as presented by the licensee's submittal, we agree with the licensee's conclusion that one MSSV is sufficient for RCS heat removal during hot shutdown conditions one hour following a reactor trip.

Since the proposed TS 3.4.1.2 requires two operable MSSVs per steam generator during plant operation when PCS temperature is between 250°F and that corresponding to hot shutdown conditions, with the reactor subcritical for at least one hour, we consider that the proposed TS provides ample margin for relieving capacity.

We have also reviewed the licensee's modified bases for proposed TS 3.4.1.2 and find them acceptable.

The proposed change is supported by the licensee's analysis since adequate steam relieving capacity will still be maintained during heatup and hot shutdown operations. Therefore, we conclude that the proposed change of TS 3.4.1.2 is acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 9, 1987

Principal Contributor:
C. Liang