

March 20, 1990

Mr. Henry D. Hukill, Vice President
and Director - TMI-1
GPU Nuclear Corporation
P.O. Box 480
Middletown, Pennsylvania 17057

Dear Mr. Hukill:

SUBJECT: THREE MILE ISLAND UNIT 1 - TECHNICAL SPECIFICATION CHANGE REQUEST
NO. 199 - STEAM GENERATOR TUBE INSERVICE INSPECTION

The Commission has forwarded the enclosed "Notice of Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing" to the Office of the Federal Register for publication.

This notice relates to your March 12, 1990 application to amend the TMI-1 Technical Specifications to provide specific inservice inspection requirements in the event of a primary-to-secondary leak.

Sincerely,

signed by Alan Wang for

Ronald W. Hernan, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
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Mr. Henry D. Hukill
GPU Nuclear Corporation

Three Mile Island Nuclear Station,
Unit No. 1

cc:

G. Broughton
O&M Director, TMI-1
GPU Nuclear Corporation
Post Office Box 480
Middletown, Pennsylvania 17057

Francis I. Young
Senior Resident Inspector (TMI-1)
U.S.N.R.C.
Post Office Box 311
Middletown, Pennsylvania 17057

Richard J. McGoey
Manager, PWR Licensing
GPU Nuclear Corporation
100 Interpace Parkway
Parsippany, New Jersey 07054

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

C. W. Smyth
TMI-1 Licensing Manager
GPU Nuclear Corporation
Post Office Box 480
Middletown, Pennsylvania 17057

Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 525
1700 Rockville Pike
Rockville, Maryland 20852

Ernest L. Blake, Jr., Esq.
Shaw, Pittman, Potts & Trowbridge
2300 N Street, N.W.
Washington, D.C. 20037

Governor's Office of State Planning
and Development
ATTN: Coordinator, Pennsylvania
State Clearinghouse
Post Office Box 1323
Harrisburg, Pennsylvania 17120

Sally S. Klein, Chairperson
Dauphin County Commissioner
Dauphin County Courthouse
Front and Market Streets
Harrisburg, Pennsylvania 17120

Thomas M. Gerusky, Director
Bureau of Radiation Protection
Pennsylvania Department of
Environmental Resources
Post Office Box 2063
Harrisburg, Pennsylvania 17120

Kenneth E. Witmer, Chairman
Board of Supervisors
of Londonderry Township
25 Roslyn Road
Eilzabethtown, PA 17022

UNITED STATES NUCLEAR REGULATORY COMMISSIONGPU NUCLEAR CORPORATIONDOCKET NO. 50-289NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR-50, issued to GPU Nuclear Corporation (the licensee), for operation of Three Mile Island, Unit 1 located in Dauphin County, Pennsylvania.

By application dated March 12, 1990, the licensee requested that the Technical Specification (TS) requirements governing inservice inspection of the Once Through Steam Generators (OTSGs) be amended. Specifically, the licensee requested modification to the eddy current testing (ECT) required following excessive primary-to-secondary leakage in tubes located in the area of the OTSG defined as the "lane wedge" area. This area is described as the tubes in rows 73 through 79 adjacent to the open inspection lane, and tubes between and on lines drawn from tube 66-1 to tube 75-15 and from 86-1 and 77-15. The present TS do not make a distinction for tubes located in this area. However, operating experience for OTSGs has indicated that tubes in these areas are more subject to certain types of failures, including environmentally assisted high cycle fatigue (HCF) failures, than tubes located elsewhere in the OTSG. Of approximately 16,000 tubes in the OTSG, 419 are located in the lane wedge area. The present TS, following a tube leak,

requires a 6% randomly selected sample (approximately 950 tubes) which would not necessarily include more than about 25 tubes in the lane wedge area. Because of the history of failures of lane wedge tubes at other Babcock and Wilcox plants, the licensee proposes concentrating ECT inspections in this area following failure in the upper portion of a lane wedge tube.

The licensee's application is consistent with a TS amendment issued by the NRC in 1981 for Oconee Units 1, 2, and 3. It is also consistent with guidelines recently issued by the Electric Power Research Institute but not yet fully endorsed by the NRC. The application for a TS amendment was requested by the NRC staff following a lane wedge tube leak at TMI-1 on March 6, 1990. The staff is issuing this notice and reviewing the licensee's application under exigent circumstances. The staff issued a waiver of compliance on March 14, 1990 to allow plant restart following leak repairs and while the application is being processed. The post-leak testing performed by the licensee following repairs is consistent with what would be required by the amended TS. In addition the licensee conducted two types of visual leak checks to ensure only one tube was responsible for primary-to-secondary leakage. The licensee did not request emergency treatment of the amended application; the staff does not believe that an emergency situation exists. However, the staff does believe that the amendment should be issued promptly.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission must make a proposed determination that the amendment request involves no significant hazards considerations. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The licensee has determined that the Technical Specifications Change Request involves no significant hazards considerations as defined in 10 CFR 50.92. That determination is as follows:

1. Operation of the facility in accordance with the proposed amendment would not involve a significant increase in the probability of occurrence or the consequences of an accident previously evaluated. The proposed amendment limits the unscheduled inservice inspection to the leaking steam generator following primary-to-secondary leakage through the steam generator tubes which exceeded Technical Specification limits. The proposed amendment also limits this unscheduled inspection to the lane wedge area when the leaking tube is located in this area. The design basis accidents related to this change are accidents related to steam generator tube integrity. The probability of occurrence or the consequences of a steam generator tube rupture accident, or a main steam line break accident, which assumes a 1 gallon per minute (gpm) primary-to-secondary leak rate, are not increased since adequate assurance of steam generator tube integrity is maintained by the proposed change. Limiting the unscheduled inservice inspection to the affected steam generator has no adverse effect on the adequacy of steam generator tube integrity. Limiting the unscheduled inservice inspection to the tubes in the lane wedge area when the leaking tube is in this area enhances plant safety by identifying potential additional tubes which may be experiencing similar wear, corrosion, or fatigue. Appropriate corrective actions are taken to prevent further degradation. The proposed change has no effect on the inspection methods or acceptance criteria; nor does it reduce the effectiveness of the overall unscheduled steam generator tube inspection program. Therefore, this change does not increase the probability of occurrence or the consequences of an accident previously evaluated.

2. Operation of the facility in accordance with the proposed amendment would not create the possibility of a new or different kind of accident from any accident previously evaluated. The proposed amendment limits the unscheduled inservice inspection to the leaking steam generator following primary-to-secondary leakage through the steam generator tubes which has exceeded Technical Specification limits. The proposed amendment also limits the unscheduled inspection to the lane wedge area when the leaking tube is located in this area. The proposed change has no affect on the inspection methods, nor does it reduce the effectiveness of the overall unscheduled steam generator tube inspection program. The proposed changes are related to steam generator tube integrity and tube rupture accidents only, which have been analyzed previously. Therefore, the change has no effect on the possibility of creating a new or different kind of accident from any accident previously evaluated.

3. Operation of the facility in accordance with the proposed amendment would not involve a significant reduction in the margin of safety. The proposed amendment limits the unscheduled inservice inspection to the leaking steam generator following primary-to-secondary leakage through the steam generator tubes which exceeded Technical Specification limits. The proposed amendment also limits the unscheduled inservice inspection to the lane wedge area when the leaking tube is located in this area. Adequate assurance of steam generator tube integrity is maintained and plant safety is enhanced by identifying potential additional tubes which may be experiencing similar wear, corrosion, or fatigue in the area which is susceptible to such degradation. Appropriate corrective actions are taken to prevent further degradation. Performing a 100% inspection of the lane wedge area tubes following a tube leak in excess of the Technical Specification limits enhances plant safety by identifying tubes with similar degradation. The proposal has no effect on the inspection methods or acceptance criteria, nor does it reduce the effectiveness of the overall unscheduled steam generator tube inspection program. Therefore, it is concluded that operation of the facility in accordance with the proposed amendment does not involve a significant reduction in a margin of safety.

The staff has reviewed the licensee's no significant hazards consideration determination analysis and agrees with its conclusion. Therefore, the staff proposes to determine that the application for amendment does not involve a significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 15 days after the date of publication of this

notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

Written comments may be submitted by mail to the Regulatory Publications Branch, Division of Freedom of Information and Publications Services, Office of Administration, U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room P-223, Phillips Building, 7920 Norfolk Avenue, Bethesda, Maryland, from 7:30 a.m. to 4:15 p.m. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW, Washington, D.C. The filing of requests for hearing and petitions for leave to intervene is discussed below.

By April 23, 1990 , the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for hearing and a petition for leave to intervene. Requests for a hearing and petitions for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, N.W., Washington, D.C. 20555 and at the Local Public Document Room located at the Government Publications Section, State Library of Pennsylvania, Walnut Street and Commonwealth Avenue, Box 1601, Harrisburg, Pennsylvania 17105.

If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement

of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendments under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If the amendment is issued before the expiration of 30-days, the Commission will make a final determination on the issue of no significant hazards consideration. If a hearing is requested, the final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves significant hazards considerations, any hearing held would take place before the issuance of any amendment.

Normally, the Commission will not issue the amendment until the expiration of the 15-day notice period. However, should circumstances change during the notice period, such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 15-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish a notice of issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, N.W., Washington, D.C., by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 325-6000 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to John F. Stolz: petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be

sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and to Ernest L. Blake, Jr., Esquire, Shaw, Pittman, Potts & Trowbridge, 2300 N Street, Washington, D.C. 200317, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated March 12, 1990, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, N.W., Washington, D.C. 20555, and at the Local Public Document Room located at the Government Publications Section, State Library of Pennsylvania, Walnut Street and Commonwealth Avenue, Box 1601, Harrisburg, Pennsylvania 17105.

Dated at Rockville, Maryland, this 15th day of March 1990.

FOR THE NUCLEAR REGULATORY COMMISSION

Alan Wang

Alan B. Wang, Acting Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation