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Docket No. 50-289

Mr. Henry D. Hukill, Vice President
and Director - TMI-1
Metropolitan Edison Company
P. O. Box 480
Middletown, Pennsylvania 17057

AEOD
Hornstein
EBTackwood
JWetmore
RDiggs
CMiles
HDenton



Dear Mr. Hukill:

The Commission has issued the enclosed Amendment No. 62 to Facility Operating License No. DPR-50 for Three Mile Island Nuclear Station, Unit No. 1 (TMI-1). The amendment consists of changes to the Technical Specifications in response to your request dated April 28, 1977 (GLO554; TSCR 53), and staff discussions.

The amendment revises the Technical Specifications for TMI-1 regarding the nomenclature describing the analyses of radioactive contamination of the secondary coolant (TSCR 53.3) and a clarification on the use of the shutdown bypass switch associated with each reactor protection channel (TSCR 53.5). Other items you requested have been resolved by issuances of past amendments except for the administrative requirements concerned with review of procedures (TSCR 53.1). This matter will be addressed upon receipt of your revision to administrative section of the Technical Specifications.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by
Robert W. Reid

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 62
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next 3 pages

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Off conditional on change noted on Federal Register Notice

OFFICE	ORB#4:DL	ORB#4:DL	C-ORB#4:DL	AD-OR:DL	OELD		
SURNAME	EHylton	DDiIanni/cb	RReid	TNovak	J.R. GRAY		
DATE	2/10/80	2/21/80	2/26/80	2/27/80	2/25/80		



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 11, 1981

Docket No. 50-289

Mr. Henry D. Hukill, Vice President
and Director - TMI-1
Metropolitan Edison Company
P. O. Box 480
Middletown, Pennsylvania 17057

Dear Mr. Hukill:

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The amendment revises the Technical Specifications for TMI-1 regarding the nomenclature describing the analyses of radioactive contamination of the secondary coolant (TSCR 53.3) and a clarification on the use of the shutdown bypass switch associated with each protection channel (TSCR 53.5). Other items you requested have been resolved by issuances of past amendments except for the administrative requirements concerned with review of procedures (TSCR 53.1). This matter will be addressed upon receipt of your revision to administrative section of the Technical Specifications.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A handwritten signature in cursive script, appearing to read "Robert W. Reid".

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 62
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next 3 pages

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Metropolitan Edison Company

- 1 -

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER AND LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

DOCKET NO. 50-289

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 62
License No. DPR-50

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Metropolitan Edison Company, Jersey Central Power and Light Company and Pennsylvania Electric Company (the licensees), dated April 28, 1977 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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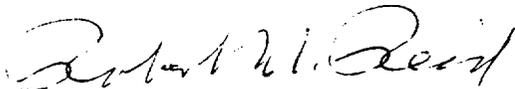
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-50 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.62, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 11, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 62

FACILITY OPERATING LICENSE NO. DPR-50

DOCKET NO. 50-289

Revise Appendix A as follows:

Remove Pages

Insert Pages

3-27

3-27

4-9

4-9

4-10

4-10

The changes on the revised pages are shown by marginal lines.

3.5 INSTRUMENTATION SYSTEMS

3.5.1 OPERATIONAL SAFETY INSTRUMENTATION

Applicability

Applies to unit instrumentation and control systems.

Objective

To delineate the conditions of the unit instrumentation and safety circuits necessary to assure reactor safety.

Specifications

- 3.5.1.1 The reactor shall not be in a startup mode or in a critical state unless the requirements of Table 3.5-1, Column 'A' and 'B' are met.
- 3.5.1.2 For on-line testing or in the event of a protection instrument or channel failure, a key operated channel bypass switch associated with each reactor protection channel will be used to lock the reactor trip module in the untripped state as indicated by a light. Only one channel shall be locked in this untripped state at any one time. Unit operation at rated power shall be permitted to continue with Table 3.5-1, Column "A". Only one channel bypass key shall be kept in the control room.
- 3.5.1.3 In the event the number of protection channels operable falls below the limit given under Table 3.5-1, Column "A", operation shall be limited as specified in Column "C".
- 3.5.1.4 The key operated shutdown bypass switch associated with each reactor protection channel shall not be used during reactor power operation (except for required maintenance or testing).
- 3.5.1.5 During startup when the intermediate range instruments come on scale, the overlap between the intermediate range and the source range instrumentation shall not be less than one decade.
- 3.5.1.6 In the event that one of the trip devices in either of the sources supplying power to the control rod drive mechanisms fails in the untripped state, the power supplied to the rod drive mechanisms through the failed trip device shall be manually removed within 30 minutes. The condition will be corrected. The remaining trip device shall be tested within eight hours. If the condition is not corrected and the remaining trip devices are not tested within the 8 hour period, the reactor shall be placed in the hot shutdown condition within an additional 4 hours.

Bases

Every reasonable effort will be made to maintain all safety instrumentation in operation. A startup is not permitted unless three power range neutron instrument channels and two channels each of the following are operable: four

TABLE 4.1-3

MINIMUM SAMPLING FREQUENCY

<u>Item</u>	<u>Check</u>	<u>Frequency</u>
1. Reactor Coolant	a. Radio-Chemical Analysis(1)	Monthly
	b. E determination (2)	Semiannually
	c. 15 Min. Gross Degassed Beta-Gamma Activity (1)	5 times/week when Tav _g is greater than 200°F
	d. Tritium Radioactivity	Monthly
	e. Chemistry (Cl, F and O ₂)	5 times/week when Tav _g is greater than 200°F
	f. Boron Concentration	2 times/week
2. Borated Water Storage Tank Water Sample	Boron Concentration	Weekly and after each makeup when reactor coolant system pressure is greater than 300 psig or Tav is greater than 200°F
3. Core Flooding Tank Water Sample	Boron Concentration	Monthly and after each makeup when RCS pressure is greater than 700 psig
4. Spent Fuel Pool Water Sample	Boron Concentration	Monthly and after each makeup
5. Secondary Coolant	a. Gross Activity	Weekly when reactor coolant system pressure is greater than 300 psig or Tav is greater than 200°F
	b. Iodine Analysis (3)	
6. Boric Acid Mix Tank or Reclaimed Boric Acid Tank	Boron Concentration	Twice weekly
10. Sodium Hydroxide Tank	Concentration	Quarterly and after each makeup

TABLE 4.1-3 (Continued)

<u>Item</u>	<u>Check</u>	<u>Frequency</u>
11. Sodium Thiosulphate Tank	Concentration	Quarterly and after each makeup
12. Condenser Partition Factor	I^{131} Partition Factor	Once if primary/secondary leakage develops, i.e.: Gross Beta-Gamma on secondary side of OTSG is greater than 2×10^{-8} micro curies per cc and evidence of fission products is present

- (1) When radioactivity level is greater than 10 percent of the limits of Specification 3.1.4, the sampling frequency shall be increased to a minimum of 5 times per week.
- (2) \bar{E} determination will be started when the 15 minute gross degassed beta-gamma activity analysis indicates greater than 10 $\mu\text{Ci/ml}$ and will be redetermined each 10 $\mu\text{Ci/ml}$ increase in the 15 minute gross degassed beta-gamma activity analysis. A radio chemical analysis for this purpose shall consist of a quantitative measurement of 95 percent of radionuclides in reactor coolant with half lives of >30 minutes.
- (3) When the gross activity increases by a factor of two above background, an iodine analysis will be made and performed thereafter when the gross activity increases by 10 percent.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 62 TO FACILITY OPERATING LICENSE NO. DPR-50

METROPOLITAN EDISON COMPANY
JERSEY CENTRAL POWER AND LIGHT COMPANY
PENNSYLVANIA ELECTRIC COMPANY

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-289

Introduction

By letter dated April 28, 1977 (GOL 0554, TSCR 53), Metropolitan Edison Company (the licensee) requested five changes to the Three Mile Island Nuclear Station, Unit No. 1 (TMI-1) Technical Specifications (TS).

These changes address the administrative requirements concerning reviews of procedures, equipment or system changes (TSCR 53.1), the frequency of successive power mapping (TSCR 53.2), the nomenclature describing the analysis of radioactive contamination of the secondary coolant (TSCR 53.3), the reporting requirements for the ring girder surveillance inspection (TSCR 53.4) and the status of the shutdown bypass switch associated with each reactor protection channel during power operation (TSCR 53.5).

This Safety Evaluation Report addresses Met Ed's requests on the nomenclature describing the analysis of radioactive contamination of secondary coolant (TSCR 53.3) and the status of the shutdown bypass switch associated with each reactor protection channel (TSCR 53.5). This report does not address three of the requests in Met Ed's submittal for the following reasons:

1. Administrative requirements concerned with reviews of procedures (53.1).

This request is being addressed as part of the general revision to the Administrative Section 5 of the TS. The general revision of Section 6 is necessary as the result of the formation of General Public Utilities Nuclear Group (GPUNG) (Note Amendment No. 93).

2. Frequency of successive power mapping (53.2).

This request permits an increase in the intervals between successive power mappings from 10 EFPD to 30 EFPD. The TS has been addressed as part of Amendment No. 29 (letter dated April 22, 1977) and there is no need to address this matter as part of this evaluation.

3. Reporting requirements for the ring girder surveillance inspection (53.4).

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This matter had been addressed as part of Amendment No. 59 (letter dated October 31, 1980) and there is no need to reconsider the reporting requirement as part of this evaluation.

Discussion and Evaluation

I. Nomenclature Describing the Analysis of Radioactive Contamination -Secondary Coolant (TSCR 53.3).

The proposed TS change request (TSCR 53.3) amends the type of radiochemical analysis used in measuring secondary system coolant radioactivity. Presently, TMI-1 incorporates a "15 minute gross degassed β - γ " analysis for secondary coolant radioactivity which requires sample collection, preparation and analysis be completed in approximately 15 minutes. Because of both logistics and analytical sensitivity, the licensee is requesting that a "gross activity" measurement be incorporated for future secondary coolant activity measurement.

The proposed change pertains specifically to the analytical method used in measuring radioactivity in the secondary system. The standardized TSs require a "gross activity" measurement for determining the presence of a primary to secondary (steam generator) leak. The licensee's TS requires a low level of detectability (LLD) of 2×10^{-8} $\mu\text{Ci}/\text{cc}$. Because of the state-of-the-art in analytical hardware this specification will require both sample pre-concentration and longer count times to attain analytical detection at the LLD limit. Therefore, a functional need exists for the proposed TS change because it is difficult with present state-of-the-art technology to perform this analysis at this LLD limit within a 15 minute period.

Since neither the TS for secondary coolant activity (I^{131} - $1.0 \mu\text{Ci}/\text{cc}$) nor the frequency of analysis is changed, the intent of early detection of steam generator leakage remains intact. In regard to effluent TSs, there is no associated impact with the proposed secondary coolant radiochemical analysis change. On this basis, we conclude that the level of safety is not reduced and therefore, changing the 15 minute gross degassed β - γ to gross activity is acceptable.

II. Status of the Shutdown Bypass Switch Associated with each Protection Channel (TSCR 53.5)

The proposed TS change (TSCR 53.5) requests a change to the wording of TS 3.5.1.4 from: "The key operated shutdown bypass switch associated with each reactor protection channel shall not be used during reactor power operation", to: "The key operated shutdown bypass switch associated with each reactor protection channel shall not be used during reactor power operation except for required maintenance or testing".

The proposed change consists of adding the phrase "Except for required maintenance or testing" to the end of the specification. This proposed change

allows the shutdown bypass switch associated with each reactor protection channel to be placed in a bypass position during reactor power operation for testing and maintenance purposes.

Each reactor protection channel is provided with two key-operated bypass switches, consisting of a channel (manual) bypass switch and a shutdown bypass switch.

The channel bypass switch enables a protection channel to be bypassed taking the protection channel out of service for testing and maintenance purposes. Actuation of this switch initiates a visual alarm on the main console which remains in effect so long as the channel is bypassed. During testing the system operates in 2-out-of-3 coincidence. The channel bypass switches for redundant protection channels are wired such that if one bypass switch is placed in the bypass position, then placing any other redundant channel in the bypass mode will have no effect. Thus by design, two or more protection channels cannot be bypassed simultaneously by the accidental actuation of more than one channel bypass switch. It should be noted, that when a protection channel is bypassed by the channel bypass switch, all safety signals that normally pass through the protection channel are blocked.

The shutdown bypass switch enables the power imbalance flow, power-pump, low pressure, and pressure-temperature trips to be bypassed when bringing the plant up or down in pressure, and allows control rod drive tests to be performed after the reactor has been shutdown and depressurized below the low reactor coolant pressure trip point. An additional bistable is employed in the shutdown bypass circuits for each reactor protection channel to trip the channel if the reactor coolant pressure exceeds 1,720 psig when the shutdown bypass switch in a protection channel is in the "bypass" position.

Item 8 in TS Table 4.1-1, requires a monthly test of the reactor coolant high pressure bistable to verify it operates as designed. In order to perform the test, the channel shutdown bypass switch must be placed in the bypass position to verify that a trip signal from the shutdown bypass bistable will trip the channel. The proposed change eliminates a conflict existing between two requirements in the TS, that is, Item 8 in Table 4.1-1 and TS Item 3.5.1.4 which requires the shutdown bypass switch not be placed in the bypass position during power operation. The proposed change will in no way reduce the level of safety because of redundancy of the channels that cannot be bypassed while a single bypass shutdown switch is actuated. In addition, the protection channel under test is placed in the bypass mode by the actuation of both the channel bypass switch and the shutdown bypass switch. The proposed change will permit testing and or any required maintenance of the reactor protection channel shutdown bypass bistables to ensure their proper operation without any potential ambiguity.

On this basis, we conclude that the change to the TSs allowing the use of the shutdown bypass switch during maintenance or testing to assure proper operation of the reactor protection channel shutdown bypass bistables is acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: February 11, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-289METROPOLITAN EDISON COMPANYJERSEY CENTRAL POWER AND LIGHT COMPANYPENNSYLVANIA ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 62 to Facility Operating License No. DPR-50, issued to Metropolitan Edison Company, Jersey Central Power and Light Company, and Pennsylvania Electric Company (the licensees), which revised Technical Specifications for operation of the Three Mile Island Nuclear Station, Unit No. 1 (the facility) located in Dauphin County, Pennsylvania. The amendment is effective as of its date of issuance.

The amendment revises the Technical Specifications for the facility regarding the nomenclature describing the analyses of radioactive contamination of the secondary coolant and modifies the procedures on the use of the key-operated shutdown bypass switch associated with each reactor protection channel during reactor power operation to allow its use for required maintenance or testing.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

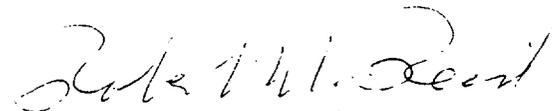
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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated April 28, 1977, (2) Amendment No. 62 to License No. DPR-50, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, DC 20555, and at the Government Publications Section, State Library of Pennsylvania, Box 1601 (Education Building), Harrisburg, Pennsylvania 17126. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 11th day of February 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

FEB 18 1981