

December 21, 1984

DMB 016

Docket No. 50-289

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Mr. Henry D. Hukill
Vice President
GPU Nuclear Corporation
P. O. Box 480
Middletown, Pennsylvania 17057

Dear Mr. Hukill:

SUBJECT: LICENSE AMENDMENT NO. 103, STEAM GENERATOR TUBE REPAIRS AND RETURN TO OPERATION, THREE MILE ISLAND NUCLEAR STATION, UNIT 1 (TMI-1)

The Commission has issued the enclosed Amendment No. 103 to Facility Operating License No. DPR-50 for TMI-1. This amendment permits the return of the steam generators to operation in response to your amendment request of May 9, 1983, and is in accordance with the Initial Decision of the Atomic Safety and Licensing Board dated October 31, 1984.

Your letter of May 9, 1983, requested a change in Technical Specifications (TS) 4.19 which would permit operation of TMI-1 after repair of the steam generators by any method other than plugging, provided that the repair method was approved by the NRC. It further requested approval of the kinetic expansion process to permit both a non-nuclear heatup of the plant using pump heat for pre-critical testing and subsequent operation using the repaired steam generators.

Amendment No. 86, issued on August 25, 1983, addressed a portion of your request of May 9, 1983, by modifying TS 4.19 to include only the kinetic expansion repair process as an alternative to plugging defective tubes, only for the period of pre-critical steam generator hot functional testing, and permitted such testing. The Safety Evaluation (SE) accompanying that amendment included a final no significant hazards consideration determination related to such testing only.

Similarly, Amendment No. 91, issued on April 9, 1984, further modified TS 4.19 to cover the total period of pre-critical (non-nuclear) hot functional testing of the plant. The SE accompanying that amendment included a final no significant hazards consideration determination only for hot functional testing.

In response to the FEDERAL REGISTER notice of May 31, 1983, a hearing was requested on this matter. Intervenors and contentions were admitted in the Board's order of November 29, 1983. After summary disposition motions by the licensee and the staff, the Board dismissed many contentions and one of the

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intervenors, and a hearing was held on July 16-18, 1984 on specific issues identified by the Board. The Board published its Initial Decision on October 31, 1984, authorizing the Director of Nuclear Reactor Regulation to issue the instant amendment subject to satisfaction of the conditions previously identified by the staff in NUREG-1019 as modified by Supplement 1 thereto, and of two additional conditions imposed by the Board in its decision.

The first of these conditions, dealing with the condenser offgas radiation monitor RM-A5, is incorporated into the revised Technical Specifications included in this amendment. Your letter of November 19, 1984, indicating the alternative you prefer of the two offered by the Board, was considered in the revision to the Technical Specifications. The staff has concluded that your proposed implementation of that alternative is acceptable and constitutes a suitably equivalent system (to RM-A-5) of comparable sensitivity and response time, and that the revised Technical Specification included in the enclosed amendment conforms with the condition imposed by the Board. The second condition, which adds eddy current testing requirements in the event of extended operation below 50% power, is incorporated in the body of the amendment.

This amendment, therefore, further modifies TS 4.19 by removing restrictions as to the period of effectiveness of the acceptability of the kinetic expansion repair process as an alternative to plugging defective tubes. The amendment, subject to the license conditions noted therein, permits the return of the steam generators to operation. In conformance with the Board's order, the license is conditioned to require primary-to-secondary leakage restrictions, power ascension test program results, extended inservice inspection, and reporting on corrosion lead tests. The revised Technical Specifications incorporate the Board's condition related to RM-A5.

In Section 4.3.1 of Supplement No. 1 to NUREG-1019, we note that in the longer term, consistent with our ATOG SER requirement, we will require additional systems analyses regarding SGTR emergency procedure guidelines. Specific requirements have been forwarded to you, and you have provided additional information. Further analyses have been delayed to early 1985, which remains consistent with the delayed schedule for the B&W Owners Group.

In Section 4.3.1 of Supplement No. 1, we also note that we will require that the TMI-1 SGTR guidelines and procedures be revised prior to criticality to define alternative actions that can be employed to minimize offsite releases consistent with overall safety. By letters dated January 19, 1984, and May 17, 1984, you submitted copies of these revised documents. Our letter to you dated June 15, 1984, documents our conclusion that the procedures and guidelines adequately define such alternative actions.

In addition, in a related matter which will further limit radioactive releases in steam generator tube ruptures, you committed in your letter of November 15, 1983, to submit a Technical Specification change request in the very near future to change the RCS specific activity limits to conform to the Standard Technical Specifications. You submitted a change request including these activity limits with your letter of November 24, 1983. Staff action on this matter is proceeding separately.

Supporting bases for this amendment are contained in NUREG-1019, the staff SE covering the steam generator tube repair and return to operation, and Supplement No. 1 thereto which is transmitted with this letter. Supplement 1 had previously been made available to all parties with Board Notification BN-83-184, dated November 23, 1983.

The hearing on these matters having been completed as discussed above, the matter of a Final Determination of No Significant Hazards Consideration related to this amendment is moot, and no such determination is required or made. It is noted that responses to comments received by the staff on FEDERAL REGISTER notices on this matter which had not been previously made in the earlier final no significant hazards consideration determinations discussed above relating to Amendments 86 and 91 had been made in the draft final determination which was part of Commission Paper SECY-83-474. This document and its attachments had been transmitted to all parties with Board Notification BN-83-184 on November 23, 1983.

In NUREG-1019, as modified by Supplement No. 1 thereto, we concluded that the steam generator repair program, including the steam generator tube kinetic expansion process, is acceptable. We further conclude, based on the considerations discussed above, and consistent with the Licensing Board's Memorandum and Order (Rulings on Motions for Summary Disposition) issued June 1, 1984 and Initial Decision (Amendment to Operating License) issued October 31, 1984, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in accordance with the amended license, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

In connection with the issuance of this amendment, we prepared an "Environmental Assessment and Finding of No Significant Impact" which was sent to you separately and was published in the Federal Register on December , 1984.

A copy of the "Notice of Issuance of Amendment to Facility Operating License" is also enclosed.

Sincerely,

ORIGINAL SIGNED BY
JOHN F. STOLZ, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:

- 1. Amendment No. 103
- 2. Supplement No. 1 to NUREG-1019
- 3. Notice

cc w/enclosures:
See next page

*See previous white for concurrences.

ORB#4:DL	ORB#4DL	C-ORB#4:DL	AD:OR:DL	OELD
RIngram*	HSilver;cf	JStolz*	GLainas	MWagner*
11/29/84	12/11/84	11/29/84	12/14/84	12/4/84
D:DE	METB:DSI	AEB:DSI		
RVollmer*	WGammill*	LHulman*		
12/3/84	12/3/84	12/3/84		

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John F. Stolz, Chief
Operating Reactors Branch #4
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wanted changes to letter, New Service list, amendment notice and etc
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- 2 -

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- 3 -

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