

April 9, 1984

DMB 016

Docket No. 50-289

Mr. Henry D. Hukill, Vice
President and Director - TMI-1
GPU Nuclear Corporation
P. O. Box 480
Middletown, Pennsylvania 17057

Dear Mr. Hukill:

SUBJECT: THREE MILE ISLAND NUCLEAR STATION, UNIT 1 (TMI-1), HOT
FUNCTIONAL TESTING LICENSE AMENDMENT

The Commission has issued the enclosed Amendment No. 91 to Facility Operating License No. DPR-50 for TMI-1. This amendment permits the steam generators to be declared operable for hot functional testing using pump heat (non-nuclear) in partial response to your amendment request of May 9, 1983, and to Mr. Dieckamp's letter to Mr. Denton of July 13, 1983, and in response to Mr. Clark's letter to Mr. Denton of February 2, 1984.

Your letter of May 9, 1983, requested a change in Technical Specification (TS) 4.19 which would permit operation of TMI-1 after repair of the steam generators by methods other than plugging, provided that the repair method was approved by the NRC. It further requested approval of the kinetic expansion process to permit both a non-nuclear heatup of the plant using pump heat for pre-critical testing and subsequent operation using the repaired steam generators. Mr. Dieckamp's letter requested approval of pre-critical non-nuclear hot functional testing in advance of approving nuclear operations. Mr. Clark's letter of February 2, 1984, reiterates that request.

Amendment No. 86 modified TS 4.19 to include the kinetic expansion repair process as an alternative to plugging defective tubes, only for the period of pre-critical steam generator hot testing. This amendment further modifies TS 4.19 to cover the total period of pre-critical (non-nuclear) hot functional testing. The accompanying Safety Evaluation (SE) includes a final no significant hazards consideration determination related only to such testing with the repaired steam generators. Thus, this amendment addresses a portion of your May 9, 1983 request, and no portion of the amendment falls outside of your request.

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Mr. Henry D. Hukill

-2-

Copies of the Safety Evaluation and Notice of Issuance and Final Determination of No Significant Hazards Consideration are enclosed.

Sincerely,

"ORIGINAL SIGNED BY
JOHN F. STOLZ"

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:

- 1. Amendment No. 91 to DPR-50
- 2. Safety Evaluation
- 3. Notice

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

April 9, 1984

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DOCKET No. 50-289

MEMORANDUM FOR: Docketing and Service Branch
Office of the Secretary of the Commission

FROM: Office of Nuclear Reactor Regulation

SUBJECT: THREE MILE ISLAND NUCLEAR STATION, UNIT 1

One signed original of the *Federal Register* Notice identified below is enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (6) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Consideration of Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Order.
- Exemption.
- Notice of Granting of Relief.

Other: Amendment No. 91
Referenced documents have been provided PDR.

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Division of Licensing, ORB#4
Office of Nuclear Reactor Regulation

Enclosure:
As stated

OFFICE	ORB#4:DL						
SURNAME	RIngram;cf						
DATE	4/11/84						

OFFICIAL RECORD COPY

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GPU Nuclear Corporation

- 2 -

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ATTN: Coordinator, Pennsylvania
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER AND LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

GPU NUCLEAR CORPORATION

DOCKET NO. 50-289

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91
License No. DPR-50

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation, et al (the licensees) dated May 9, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, Facility Operating License No. DPR-50 is hereby amended as indicated below and by changes to the Technical Specifications as indicated in the attachment to this license amendment.

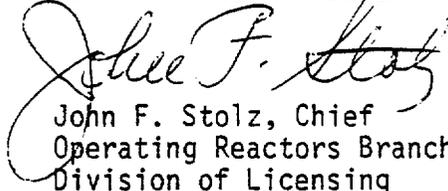
Revise paragraph 2.C.(2) to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 91, are hereby incorporated in the license. The GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 9, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 91

FACILITY OPERATING LICENSE NO. DPR-50

DOCKET NO. 50-289

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

Remove

4-81

Insert

4-81

- b. The steam generator shall be determined OPERABLE after completing the corresponding actions (removal from service by plugging, or repair by the kinetic expansion process*, of all tubes exceeding the repair limit and all tubes containing throughwall cracks) required by Table 4.19.2.

4.19.5 Reports

- a. Following the completion of each inservice inspection of steam generator tubes, the number of tubes repaired or removed from service in each steam generator shall be reported to the NRC within 15 days.
- b. The complete results of the steam generator tube inservice inspection shall be reported to the NRC within 3 months following completion of the inspection. This report shall include:
 - 1. Number and extent of tubes inspected.
 - 2. Location and percent of wall-thickness penetration for each indication of an imperfection.
 - 3. Identification of tubes repaired or removed from service.
- c. Results of steam generator tube inspections which fall into Category C-3 and require prompt notification of the NRC shall be reported pursuant to Specification 6.9.2 prior to resumption of plant operation. The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence.

Bases

The Surveillance Requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained.

*The phrase "or repair by the kinetic expansion process" shall be in effect only during the period of pre-critical hot functional testing.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NO. DPR-50

METROPOLITAN EDISON COMPANY
JERSEY CENTRAL POWER AND LIGHT COMPANY
PENNSYLVANIA ELECTRIC COMPANY
GPU NUCLEAR CORPORATION

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-289

Pre-Critical Hot Functional Test Program

1.0 Introduction

The purpose of this Safety Evaluation is to evaluate the safety of conducting non-nuclear heatup and pre-critical testing of the plant using the repaired steam generators and to make a final determination as to significant hazards considerations of such pre-critical testing.

On March 23, 1981, in CLI 81-03, the Nuclear Regulatory Commission (the Commission) authorized the licensee to perform hot functional testing using non-nuclear heat, subject to appropriate NRC staff review. The staff reviewed the licensee's hot functional test program and approved performance of such testing, which was conducted in August and September of 1981. Subsequently, the corrosion problem which occurred in late 1981 rendered the steam generators inoperable. Although the authorization to perform hot functional testing still exists, the Technical Specifications preclude any operation (including hot functional testing) above 250°F with inoperable steam generators. GPU Nuclear submitted Technical Specification Change Request No. 125 on May 9, 1983, requesting an amendment to approve repairs to defective steam generator tubes using methods other than plugging and requested approval of the use of the repaired steam generators to perform non-nuclear heatup of the plant for pre-critical testing as well as subsequent operation. A Notice of Consideration of Issuance of Amendment and Proposed No Significant Hazards Consideration Determination covering this request appeared in the FEDERAL REGISTER on May 31, 1983. Comments received on that notice did not address hot functional testing as such, but rather concentrated on the repair process and subsequent nuclear operation with the repaired steam generators.

To facilitate hot testing of the steam generators in order to confirm the adequacy of the repairs, the licensee requested, on July 13, 1983, that their May 9 request be split into two parts: 1) pre-critical (non-nuclear) hot functional testing and 2) critical operation.

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On August 25, 1983, the staff issued its Safety Evaluation (SE) on steam generator repair and return to operation (NUREG-1019), in which it concluded that the repair and test program is acceptable, that pertinent general design criteria (GDC) have been met, that the steam generator has been returned to its original licensing basis and that there is reasonable assurance that the health and safety of the public will not be endangered by operation of the plant with the repaired steam generators.

In partial response to the licensee's July 13 request, the staff also issued, on August 25, 1983, License Amendment No. 86 which permitted the repaired steam generators to be declared operable after repair by the kinetic expansion process for the period of steam generator hot testing. License Amendment No. 86 addressed only the steam generator hot testing based on the staff's understanding of the licensee's July 13 request, but did not permit resumption of the full range of hot functional testing activity. The amendment included a safety evaluation and a final determination of no significant hazards consideration, and its issuance was noticed. As noted in that no significant hazards determination, the Commonwealth of Pennsylvania expressed no concern over this action. The steam generator hot tests were satisfactorily completed and the plant returned to cold shutdown. A summary of the test results was submitted by the licensee on October 25, 1983, and the staff's evaluation of these results was included in Supplement 1 to NUREG-1019.

The licensee, in its letters dated December 1 and 21, 1983, noted its need for, and requested permission to proceed with, non-nuclear hot functional testing. In its letter of February 2, 1984, the licensee summarizes the history of its requests to complete the hot functional test program, points out that its request is encompassed by its original request of May 9, 1983, explains the purpose of the hot functional test program, and again reiterates its request for authorization to perform the remaining pre-critical hot functional test program.

2.0 Evaluation

The staff's evaluation of the steam generator repair is covered in NUREG-1019. NUREG-1019 concluded that the repair of the steam generators and the steam generator test program are acceptable and that the steam generators and the Reactor Coolant System (RCS) are within the original licensing basis. Satisfactory completion of the steam generator hot functional test confirmed these conclusions.

As in the case of steam generator hot testing, plant hot functional testing at TMI-1 will involve bringing the reactor system to operating temperatures and pressures utilizing heat from the reactor coolant pumps. The reactor will not be critical during this testing. Since the requested operational mode, plant operating conditions, the physical status of the plant, and the potential hazards are the same as for the Steam Generator

Hot Functional Testing (SGHFT) amendment, the safety evaluation considerations and conclusions for the SGHFT Amendment No. 86 also apply to this amendment.

3.0 Conclusions

3.1 Final No Significant Hazards Consideration Determination

3.1.1 State Consultation

In accordance with the Commission's regulations, consultation was held with the Commonwealth of Pennsylvania by telephone. The Commonwealth expressed no concern over early pre-critical hot functional testing, either from the standpoint of safety or of no significant hazards consideration determination.

3.1.2 Response to Comments

In response to the FEDERAL REGISTER Notice of May 31, 1983 (48 FR 24231) containing the NRC proposed no significant hazards consideration determination, seven sets of comments were received. Responses only to those comments which address the steam generator repairs and pre-critical testing in relation to a no significant hazards consideration determination were addressed in the SE supporting Amendment No. 86. The comments and the responses described in Amendment No. 86 are also applicable to this amendment since the plant conditions for pre-critical hot functional testing will be the same as for the plant conditions previously considered for the SGHFT in Amendment 86.

3.1.3 No Significant Hazards Consideration Determination

The Commission's regulations in 10 CFR 50.92 state that the Commission may make a final determination that a proposed license amendment involves no significant hazards considerations if operation of the facility in accordance with the proposed amendment would not:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) Involve a significant reduction in a margin of safety.

NUREG-1019 and the information in this SE and in the SE prepared in support of Amendment No. 86 provide the basis for evaluating this license amendment against these criteria. Since the requested operational mode, plant operating conditions, the physical status of the plant, and dose

consequences of potential accidents are the same as that considered for Amendment No. 86 (Section 3.1.3 of that SE), the staff's evaluation against the above criteria is the same, with minor word changes. Therefore, the staff concludes that:

- (1) Operation of the facility in accordance with the proposed amendment would not significantly increase the probability or consequences of an accident previously evaluated.
- (2) Operation of the facility in accordance with the proposed amendment would not create the possibility of a new or different kind of accident from any accident previously evaluated.
- (3) Operation of the facility in accordance with the proposed amendment would not involve a significant reduction in a margin of safety.

Accordingly, we conclude that the amendment to Facility Operating License No. DPR-50 permitting non-nuclear pre-critical hot functional testing with the repaired steam generators involves no significant hazards considerations.

3.2 Environmental Consideration

We have determined that this amendment does not authorize a change in effluent types or total amounts or an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that this amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

3.3 Conclusion

Based on the considerations discussed above, we conclude that non-nuclear pre-critical hot functional testing with the repaired steam generators is acceptable. We further conclude that: there is reasonable assurance that the health and safety of the public will not be endangered by operation of TMI-1 in accordance with the amended license; such activities will be conducted in compliance with the Commission's regulations; and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 9, 1984

Contributors: Harley Silver

UNITED STATES NUCLEAR REGULATORY COMMISSIONMETROPOLITAN EDISON COMPANYJERSEY CENTRAL POWER AND LIGHT COMPANYPENNSYLVANIA ELECTRIC COMPANYGPU NUCLEAR CORPORATIONDOCKET NO. 50-289NOTICE OF ISSUANCE OF AMENDMENT TOFACILITY OPERATING LICENSE AND FINAL NO SIGNIFICANT HAZARDSCONSIDERATION DETERMINATION (PARTIAL)

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 91 to Facility Operating License No. DPR-50, issued to Metropolitan Edison Company, Jersey Central Power and Light Company, Pennsylvania Electric Company, and GPU Nuclear Corporation (the licensees), which revised the Technical Specifications (TSs) for operation of the Three Mile Island Nuclear Station, Unit No. 1 (the facility), located in Dauphin County, Pennsylvania. The amendment is effective as of the date of issuance.

The amendment revises the TSs to permit the steam generators to be declared operable for pre-critical hot functional testing using pump heat (non-nuclear), thereby permitting such testing. As such, it addresses a portion of the Notice of Consideration of Issuance of Amendment and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing identified below.

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The application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing in connection with this action was published in the FEDERAL REGISTER on May 31, 1983 (48 FR 24231), and corrected June 14, 1983 (48 FR 27328). That notice covers a requested amendment which would: recognize steam generator repair techniques other than plugging, provided such techniques are approved by the Commission; approve the kinetic expansion repair technique; and authorize subsequent operation (both non-nuclear and nuclear) of the facility with the repaired steam generators. This notice addresses a portion of, and is encompassed by, that May 31 notice.

In response to the May 31 notice, requests for hearing were filed by TMIA on May 19, 1983, as amended on June 23, 1983, and by Lee, Molholt, and Aamodt on June 30, 1983, as amended on July 13, 1983. Comments were made by six other persons and the Commonwealth of Pennsylvania.

Under its regulations, the Commission may issue and make an amendment immediately effective, notwithstanding the pendency before it of a request for a hearing from any person, in advance of the holding and completion of any required hearing, where it has determined that no significant hazards consideration is involved.

-3-

The Commission has applied the standards of 10 CFR 50.92 and has made a final determination that the amendment involves no significant hazards consideration. The basis for this determination is contained in the Safety Evaluation related to this action. Accordingly, as described above, the amendment has been issued and made immediately effective and any hearing in connection with this amendment will be held after issuance. A final determination regarding significant hazards considerations on the remainder of the subject matter of the May 31 notice, i.e., nuclear operation with the repaired steam generators, has not yet been made.

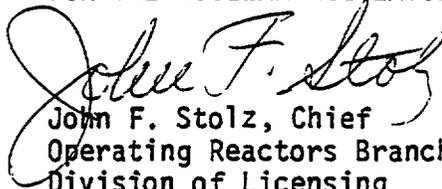
The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

For further details with respect to the action see (1) the application for amendment dated May 9, 1983, (2) Amendment No. 91 to Facility Operating License No. DPR-50, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania 17126.

A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 9th day of April 1984.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "John F. Stolz". The signature is written in dark ink and is positioned above the printed name and title.

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

Docket No. 50-289 Facility TMI-1
Licensee: GPUNC Date of application 5/9/83

1. Attach initial NSHC determination (Attachment 4) relating to this action.
2. Summary of State telephone consultation: State has no problem with this action.
3. Summary of any public comments received by telephone:
None
4. Attach any written State or public comments regarding NSHC.
5. Final determination
(XXX) The amendment request involves no significant hazards consideration.
() The amendment request involves a significant hazards consideration.

6. Basis for determination and response to comments received (Attach additi

See ref

Please place this

7. Concurr

in the docket file

a. Har

~~704~~

b. Joh

only

c. J

d. C

8. Approved:

ab *Thomas Sismant*
(Director, Division of Licensing)

4-9-84

FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

Docket No. 50-289 Facility TMI-1

Licensee: GPUNC Date of application 5/9/83

1. Attach initial NSHC determination (Attachment 4) relating to this action.
2. Summary of State telephone consultation: State has no problem with this action.
3. Summary of any public comments received by telephone:
None
4. Attach any written State or public comments regarding NSHC.
5. Final determination
(XXX) The amendment request involves no significant hazards consideration.
() The amendment request involves a significant hazards consideration.
6. Basis for determination and response to comments received. (Attach additional sheets, if necessary.)
See related Safety Evaluation.

7. Concurrences:
- | | <u>Date</u> |
|--|----------------------------------|
| a. <u><i>Harley Silver</i></u>
Harley Silver/ Jim Van Vliet
(Project Manager) | <u>3/20/84</u> 27/104 |
| b. <u><i>John F. Stoyz</i></u>
John F. Stoyz
(Branch Chief) | <u>4/1/84</u> |
| c. <u><i>J. De</i></u>
(Assistant Director) | <u>4/2/84</u> |
| d. <u><i>Richard A. Hanson</i></u>
(OELD) | <u>3/23/84</u> |

8. Approved:
Norm Sioshuit
 (Director, Division of Licensing) 4-9-84

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 PDR ADOCK 05000289
 P CF



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555
April 9, 1984

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DOCKET No. 50-289

MEMORANDUM FOR: Docketing and Service Branch
Office of the Secretary of the Commission

FROM: Office of Nuclear Reactor Regulation

SUBJECT: THREE MILE ISLAND NUCLEAR STATION, UNIT 1

One signed original of the *Federal Register* Notice identified below is enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (6) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Consideration of Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Order.
- Exemption.
- Notice of Granting of Relief.

Other: Amendment No. 91
Referenced documents have been provided PDR.

8404300261 840409
PDR ADOCK 05000289
P PDR

Division of Licensing, ORB#4
Office of Nuclear Reactor Regulation

Enclosure:
As stated

OFFICE	ORB#4:DL						
SURNAME	RIngram;cf						
DATE	4/12/84						