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Docket No. 50-289

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Mr. Henry D. Hukill, Vice President
and Director - TMI-1
GPU Nuclear Corporation
P. O. Box 480
Middletown, Pennsylvania 17057

Dear Mr. Hukill:

SUBJECT: AMENDMENT NO. 105 TO FACILITY OPERATING LICENSE NO. DPR-50

The Commission has issued the enclosed Amendment No.105 to Facility Operating License No. DPR-50 for the Three Mile Island Nuclear Station, Unit No. 1 (TMI-1). This amendment consists of changes to the Technical Specifications (TSs) in response to your submittal dated February 13, 1984.

This amendment revises the Technical Specifications to add operability and surveillance requirements for the backup incore thermocouple display channels in response to the requirements of NUREG-0737, Item II.F.2(8).

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next Monthly Federal Register Notice.

Sincerely,

*ORIGINAL SIGNED BY
JOHN F. STOLZ*

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Enclosures:

- 1. Amendment No.105
- 2. Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER AND LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

GPU NUCLEAR CORPORATION

DOCKET NO. 50-289

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 105
License No. DPR-50

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation, et al (the licensees) dated February 13, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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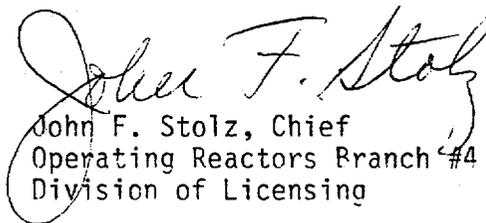
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-50 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 105, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment becomes effective 30 days after its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 5, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 105

FACILITY OPERATING LICENSE NO. DPR-50

DOCKET NO. 50-289

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3-40c

4-7a

Insert

3-40c

4-7a

TABLE 3.5-2

ACCIDENT MONITORING INSTRUMENTS

<u>FUNCTION</u>	<u>INSTRUMENTS</u>	<u>NUMBER OF CHANNELS</u>	<u>MINIMUM NUMBER OF CHANNELS</u>
1	Saturation Margin Monitor	2	1
2	Safety Valve Differential Pressure Monitor	1 per discharge line	1 per discharge line
3	PORV Position Monitor	2	1*
4	Emergency Feedwater Flow	2 per flow path	1 per flow path
5	Pressurizer Level	2	1
6	Backup Incore Thermocouple Display Channel	4 thermocouples/core quadrant	2 thermocouples/core quadrant

* With the PORV Block Valve closed in accordance with Specification 3.1.12.4.a, the minimum number of channels is zero.

TABLE 4.1-1 (Continued)

	<u>CHANNEL DESCRIPTION</u>	<u>CHECK</u>	<u>TEST</u>	<u>CALIBRATE</u>	<u>REMARKS</u>
49.	Saturation Margin Monitor	S(1)	M(1)	R	(1) When T_{ave} is greater than 525°F.
50.	Emergency Feedwater Flow Instrumentation	NA	M(1)	R	(1) When T_{ave} is greater than 250°F.
51.	Emergency Feedwater Initiation				
	a. Loss of RCP's	NA	Q(1)(2)	R	(1) When T_{ave} is greater than 250°F.
	b. Loss of both Feedwater Pumps	NA	Q(1)(2)	R	(2) Includes logic test only
52.	Backup Incore Thermocouple Display	M(1)	NA	R	(1) When T_{ave} is greater than 250°F.

S - Each Shift
D - Daily
W - Weekly
M - Monthly

T/W - Twice per week
B/M - Every 2 months
Q - Quarterly
P - Prior to each startup
If not done previous week

R - Each Refueling Period
NA - Not applicable
B/W - Every two weeks



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 105 TO FACILITY OPERATING LICENSE NO. DPR-50

METROPOLITAN EDISON COMPANY
JERSEY CENTRAL POWER AND LIGHT COMPANY
PENNSYLVANIA ELECTRIC COMPANY
GPU NUCLEAR CORPORATION

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-289

1.0 Introduction

By application dated February 13, 1984, GPU Nuclear Corporation (the licensee) requested a revision to the Technical Specifications related to the addition of operability and surveillance requirements for the backup incore thermocouples in response to the requirements of NUREG-0737, Item II.F.2(8).

2.0 Evaluation

The licensee submitted the Technical Specification change request by letter dated February 13, 1984, in response to the requirements of NUREG-0737, Item II.F.2(8), related to backup incore thermocouples. Current Technical Specifications do not address backup incore thermocouples. This change request provides Technical Specification requirements regarding operability and surveillance of the backup incore thermocouple display; namely, a display of four thermocouples per core quadrant and a minimum of two thermocouples operable per quadrant; calibration is required at each refueling period and a monthly check of that channels' capability to display four incore thermocouples per core quadrant is also required.

Based on our review, we have found that the proposed Technical Specification changes stated in Table 3.5-2 for Function 6, operability of the backup incore thermocouple display channel, and in Table 4.1-1 for Channel Description 52, surveillance of the backup incore thermocouple display, are in conformance with NUREG-0737, Item II.F.2, and are, therefore, acceptable.

3.0 Environmental Consideration

This amendment involves changes in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and in surveillance requirements. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that

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there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 5, 1985

The following NRC personnel have contributed to this Safety Evaluation:
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