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Docket No. 50-289

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Mr. Henry D. Hukill, Vice President
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Dear Mr. Hukill:

By letter dated February 4, 1985, the Commission issued Amendment No. 104 to Facility Operating License No. DPR-50 for the Three Mile Island Nuclear Station, Unit No. 1. This amendment revised the Technical Specifications (TSs) related to the allowable concentration of hydrogen and oxygen in the waste gas holdup system and the associated hydrogen/oxygen monitoring instrumentation. This amendment inadvertently omitted changes on TS pages 3-101 and 3-105a which had been made by Amendment No. 103 issued December 21, 1984. Accordingly, updated TS pages 3-101 and 3-105a are enclosed. Please incorporate these updated pages into your TSs. We apologize for any inconvenience caused by this oversight.

Sincerely,

**ORIGINAL SIGNED BY
JOHN F. STOLZ**

John F. Stolz, Chief
Operating Reactors Branch #4
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Enclosures:

Updated TS pages
3-101 and 3-105a

cc w/enclosures:
See next page

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50-289, TMI-1

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- 4.2.5 The licensee shall submit a report or application for license amendment to the NRC within 90 days after any time that Crystal River Unit No. 3 fails to maintain a cumulative reactor utilization factor of at least 65%.

The report shall provide justification for continued operation of TMI-1 with the reactor vessel surveillance program conducted at Crystal River Unit No. 3, or the application for license amendment shall propose an alternate program for conduct of the TMI-1 reactor vessel surveillance program.

For the purpose of this technical specification, the definition of commercial operation is that given in Regulatory Guide 1.16, Revision 4. The definition of cumulative reactor utilization factor is:

Cumulative reactor utilization factor - (Cumulative megawatt hours (thermal) since attainment of commercial operation at 100% power x (100)) divided by (licensed power (Mwt) x (Cumulative hours since attainment of commercial operation at 100% power)).

- 4.2.6 In addition to the reports required by Specification 4.2.4, a report shall be submitted to the NRC prior to September 1, 1982, which summarizes the first five years of operating experience with the TMI-1 integrated surveillance program performed at a host reactor. If, at the time of submission of this report, it is desired to continue the surveillance program at a host reactor, such continuation shall be justified on the basis of the attained operating experience.
- 4.2.7 A surveillance program for the pressure isolation valves between the primary coolant system and the low pressure injection system shall be as follows:
1. Periodic leakage testing^(a) at test differential pressure greater than 150 psid shall be accomplished for the valves listed in Table 3.1.6.1 for the following conditions:
 - (a) prior to achieving hot shutdown after returning the valve to service following maintenance repair or replacement work, and
 - (b) prior to achieving hot shutdown following a cold shutdown of greater than 72 hours duration unless testing has been performed within the previous 9 months.
 2. Whenever integrity of a pressure isolation valve listed in Table 3.1.6.1 cannot be demonstrated, the integrity of the other remaining valve in each high pressure line having a leaking valve shall be determined and recorded daily. In addition, the position of one other valve located in the high pressure piping shall be recorded daily.

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- (a) To satisfy ALARA requirements, leakage may be measured indirectly (as from the performance of pressure indicators) if accomplished in accordance with approved procedures and supported by computations showing that the method is capable of demonstrating valve compliance with the leakage criteria.

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Amendment No. 29, 34, 60, Order dtd.
4/20/81, 71, Corr. Ltr. dtd. 11/2/81,
Reissued 3/20/85