



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION IV  
611 RYAN PLAZA DRIVE, SUITE 400  
ARLINGTON, TEXAS 76011-8064

October 26, 2000

Gregory M. Rueger, Senior Vice  
President, Generation and Chief Nuclear Officer  
Pacific Gas and Electric Company  
Diablo Canyon Power Plant  
P.O. Box 3  
Avila Beach, CA 93424

SUBJECT: ERRATA - DIABLO CANYON SUPPLEMENTAL INSPECTION REPORT NO.  
50-275/00-13; 50-323/00-13

Dear Mr. Rueger:

Please replace the Summary of Findings page with the attached page. This change was necessary to accurately reflect how this review of a white performance indicator is required to be factored into the assessment process.

Sincerely,

*/RA/*

William B. Jones, Chief  
Project Branch E  
Division of Reactor Projects

Enclosure:  
Corrected Summary of Findings

Docket Nos.: 50-275  
50-323  
License Nos.: DPR-80  
DPR-82

cc w/enclosure:  
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Only inspection reports to the following:

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RIV:SPE:DRP/E	C:DRP/E			
GAPick;df	WBJones			
<b>/RA/</b>	<b>/RA/</b>			
10/26/00	10/26/00			

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## SUMMARY OF FINDINGS

IR 05000275-00-13, IR 05000323-00-13; on 08/21-24/2000; Pacific Gas & Electric Co.; Diablo Canyon Nuclear Power Plant, Units 1 & 2; Supplemental Inspection. No findings identified.

### **Cornerstone: Mitigating Systems**

The inspectors performed a supplemental inspection to examine a change from green to white in the Scrams With Loss of Normal Heat Removal performance indicator. This change in performance resulted from Unit 2 experiencing three scrams with loss of normal heat removal over the previous 12 quarters. Following each event, NRC had evaluated operator response, plant and equipment response, and immediate corrective actions. During this supplemental inspection, performed in accordance with Procedure 95001, the inspectors evaluated the adequacy of the root cause evaluation and long-term corrective actions for each individual event. The inspectors also evaluated the effectiveness of the licensee review into the collective events. The inspectors determined that the licensee had performed comprehensive root cause evaluations and corrective actions for each individual scram and the events collectively.

The licensee determined that one scram occurred because condensate/feedwater flow problems were exacerbated by a control circuit problem (poor design and dirty slide wire) in Valve TCV-23, generator hydrogen cold gas temperature control, combined with throttling Valve CND-2-165, steam jet air ejector outlet isolation. The licensee did not identify a definite root cause for the event initiator. Operators initiated the other two scrams because debris in the circulating water system intake had increased the differential pressure across the traveling screens above the setpoint that required them to be secured prior to being damaged. The licensee determined that the onset of ocean storms, combined with the end of the growing season (peak amounts of marine growth), established conditions that exceeded the ability of the traveling screens to remove the marine growth and remain within acceptable operating parameters. The licensee established plans to upgrade the traveling screens, formalized their process for predicting conditions affecting the ability of the intake components to remove marine growth, and initiated efforts to raise the turbine trip/reactor trip setpoint to optimize withstanding this condition yet conducting an orderly shutdown of the plants.

The inspectors concluded that the licensee addressed the Scrams With Loss of Normal Heat Removal for Unit 2 in an acceptable manner. No further evaluations are required. This is in accordance with the guidance in IMC 0305, "Operating Reactor Assessment Program." |  
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