Indiana Michigan Power Company Cook Nuclear Plant One Cook Place Bridgman, MI 49106 616-465-5901



October 19, 2000

United States Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Operating Licenses DPR-58 and DPR-74 Docket Nos. 50-315 and 50-316

**Document Control Manager:** 

In accordance with the criteria established by 10 CFR 50.73 entitled <u>Licensee Event Report System</u>, the following report is being submitted:

LER 315/1998-051-01, "Reactor Trip Signal from Manual Safety Injection Not Verified As Required By Technical Specification Surveillance"

During a review of the Technical Specification (TS) surveillance requirements for the engineered safety feature actuation system instrumentation by engineering, it was discovered that in Modes 3 and 4, the reactor trip signal from a manual initiation of a safety injection (SI) had not been verified in accordance with TS 4.3.2.1.1, Table 4.3-2, Item 9.a. This was reported in LER 315/1998-051-00.

On September 15, 2000, a review of the completed root cause for this event was performed in preparation for the submittal of the LER supplement. The review revealed that corrective actions had not been adequately implemented in response to the event reported in the LER 315/1998-051-00. As a result, another violation of the TS occurred during the June 2000 restart of Donald C Cook Nuclear Plant Unit 2. This second event is being reported under LER 316/2000-014-00. Submittal of LER 316/2000-014-00 will also provide information relevant to LER 315/1998-051-01. Therefore, LER 315/1998-051-01 is closed to LER 361/2000-014-00.

There are no commitments identified in this submittal.

Should you have any questions regarding this correspondence, please contact Mr. Wayne J. Kropp, Director Regulatory Affairs, at 616/697-5056.

Sincerely,

A. Christopher Bakken, III

Site Vice President

/mbd Attachment

TE22

J. E. Dyer, Region III D. Hahn C:

B. A. McIntyre
T. P. Noonan
R. P. Powers

R. Whale

NRC Resident Inspector Records Center, INPO

NRC Form	n 366	U.S. NUCLEAR REGULATORY COMMISSION												APPROVI	ED I	BY OMB NO. 31	50-0104	EXP	RES 06	/30/2001	
(6-1998)  LICENSEE EVENT REPORT (LER)  (See reverse for required number of digits/characters for each block)											ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-001, AND TO THE PAPERMORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503										
FACILITY	FACILITY NAME (1)														(ET	NUMBER (2)	PAGE (3)				
Cook Nuclear Plant Unit 1												05	500	00-315	1 of 1						
TITLE (4)																	•				
Re	eactor	Trip Si	gnal fr	om I	Manı	ual Safe	ety Ir	njection	Not	Veri	ified	as Re	qu	ired by	Te	echnical Sp					
EVE	NT DAT	E (5)			LER NUMBER (6)					REPORT DAT				ΓE (7)		OTHER FACILITIES INVOLVE					
MONTH	DAY	YEAR	YEA	SEQUENTIAL YEAR NUMBER				REVISION NUMBER		MONTH		DAY		YEAR		FACILITY NAME Cook Unit 2			05000-316		
11	22	1998	1			051		01		10		19		2000		FACILITY NAME			DOCKET NUMBER		
OPERA		5			PORT IS SUBMITTED PURSUAL					NT TO THE REQUIR			IRE	MENTS	OF	10 CFR §: (C					
MODE	` '								_	20.2203(a)(2)(v)				X 50.73(a)(2)(							
POWER		00	<b>⊢</b>	20.2203(a)(1)					20.2203(a)(3)(i)					50.73(a)(2)(		·		a)(2)(x)			
LEVEL	. (10)			20.2203(a)(2)(i)						20.2203(a)(3)(ii)						50.73(a)(2)(i		73.71			
				20.2203(a)(2)(ii)				20.2203(a)(4)				)				50.73(a)(2)(iv)		OTHER			
				20.2203(a)(2)(iii)					50.36(c)(1)					50.73(a)(2)			Cocción in A			ract balow	
	<u> 3 - E. H.dr</u>			20.2203(a)(2)(iv)						50.36(c)(2)				50.73(a)					NRC Form 366A		
							LIC	CENSEE	CON	ITACI	r FO	RTHIS	LEI								
NAME															TELEPHONE NUMBER (Include Area Code)						
Rhonda Allen, Regulatory Compliance  COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIPTIONS  COMPLETE ONE COMPONENT FAILURE DESCRIPTIONS  COMP															(616) 465-5901 X2027						
	<u> </u>	OMPL	ETE C	NE	LINE	FOR	EAC	H COM	IPO	NEN	T F	<u> AILUR</u>	RE	DESC	RIE	SED IN THIS	S REPO	RT (1	(3)		
CAUSE	SYSTE	м со	MPONEN	<u>,                                     </u>	MANUFACTURER		R	EPORTABLE TO EPIX			CAU	ISE	SY	SYSTEM		OMPONENT	MANUFAC	MANUFACTURER		PORTABLE TO EPIX	
					·																
	SUPPLEMENTAL REPORT EXPECTED (14)													EXPECTED			MONTH	DAY YE		YEAR	
Y	YES X												SUBMISSION				1				

Abstract (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)
This supplement is submitted to close out this LER to LER 316/2000-014-00

(If Yes, complete EXPECTED SUBMISSION DATE).

On November 22, 1998, during a review of Technical Specification (TS) surveillance requirements for the engineered safety feature actuation system instrumentation by engineering, it was discovered that in Modes 3 and 4 the reactor trip signal from a manual initiation of a safety injection (SI) had not been verified in accordance with TS 4.3.2.1.1, Table 4.3-2, Item 9.a. The TS requires the SI tests be performed for Modes 1, 2, 3 and 4 on a refueling frequency. Normally during preparations for unit restart from refueling, the reactor trip breakers are required to be open in Modes 3 and 4 for other TS surveillance testing. The tripping of the reactor trip breakers from manual SI initiation cannot be verified with the reactor trip breakers open. Therefore the reactor trip portion of the manual SI test has not normally been performed for Modes 3 and 4. An interim LER was submitted in accordance with 10CFR50.73(a)(2)(i)(B), for any condition prohibited by the plant's TS.

**DATE (15)** 

On September 15, 2000, a review of the completed root cause for this event was performed in preparation for the submittal of the LER supplement. The review revealed that corrective actions had not been adequately implemented in response to the event reported here. As a result, another violation of the TS occurred during the June 2000 restart of Donald C Cook Nuclear Plant Unit 2. LER 315/1998-051-01 is therefore closed out to LER 316/2000-014-00. Submittal of LER 316/2000-014-00 will provide information relevant both to LER 315/1998-0051-01 and the new event.