

JAFNPP

1.1 FUEL CLADDING INTEGRITY

Applicability:

The Safety Limits established to preserve the fuel cladding integrity apply to those variables which monitor the fuel thermal behavior.

Objective:

The objective of the Safety Limits is to establish limits below which the integrity of the fuel cladding is preserved.

Specifications:

- A. Reactor Pressure > 785 psig and Core Flow > 10% of Rated

The existence of a minimum critical power ratio (MCPR) less than 1.09 shall constitute violation of the fuel cladding integrity safety limit, hereafter called the Safety Limit. An MCPR Safety Limit of 1.10 shall apply during single-loop operation.

2.1 FUEL CLADDING INTEGRITY

Applicability:

The Limiting Safety System Settings apply to trip settings of the instruments and devices which are provided to prevent the fuel cladding integrity Safety Limits from being exceeded.

Objective:

The objective of the Limiting Safety System Settings is to define the level of the process variables at which automatic protective action is initiated to prevent the fuel cladding integrity Safety Limits from being exceeded.

Specifications:

- A. Trip Settings

The limiting safety system trip settings shall be as specified below:

1. Neutron Flux Trip Settings

- a. IRM - The IRM flux scram setting shall be set at $\leq 120/125$ of full scale.

(A) ROUTINE REPORTS (Continued)4. CORE OPERATING LIMITS REPORT

- a. Core operating limits shall be established prior to startup from each reload cycle, or prior to any remaining portion of a reload cycle for the following:
- The Average Planar Linear Heat Generation Rates (APLHGR) of Specification 3.5.H;
 - The Minimum Critical Power Ratio (MCPR) and MCPR low flow adjustment factor, K_f , of Specifications 3.1.B and 4.1.E;
 - The Linear Heat Generation Rate (LHGR) of Specification 3.5.I;
 - The Reactor Protection System (RPS) APRM flow biased trip settings of Table 3.1-1;
 - The flow biased APRM and Rod Block Monitor (RBM) rod block settings of Table 3.2-3; and
 - The Power/Flow Exclusion Region of Specification 3.5.J.

and shall be documented in the Core Operating Limits Report (COLR).

- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC as described in:
1. "General Electric Standard Application for Reactor Fuel," NEDE-24011-P-A-14, June 2000.
 2. "James A. FitzPatrick Nuclear Power Plant SAFER/GESTR - LOCA Loss-of-Coolant Accident Analysis," NEDC-31317P, Revision 2, April 1993.
 3. "BWR Owners' Group Long-term Stability Solutions Licensing Methodology," NEDO-31960-A, June 1991.
 4. "BWR Owners' Group Long-term Stability Solutions Licensing Methodology," NEDO-31960-A, Supplement 1, March 1992.