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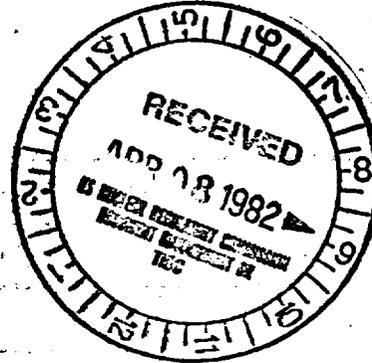
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Docket No. 50-289

Mr. Henry D. Hukill, Vice President
and Director - TMI-1
GPU Nuclear Corporation
P. O. Box 480
Middletown, Pennsylvania 17057



Dear Mr. Hukill:

The Commission has issued the enclosed Amendment No. 76 to Facility Operating License No. DPR-50 for Three Mile Island Nuclear Station, Unit No. 1 (TMI-1). The amendment consists of changes to the Technical Specifications (TSs) in response to your request dated February 25, 1982 (82-031, TSCR No. 111).

The amendment revises the TSs for TMI-1 to defer until Cycle 5 criticality, tests to verify the efficiency of HEPA filters and charcoal adsorbers in the filter systems for the Reactor Building purge exhaust, the Control Room emergency ventilation system, and the Auxiliary and Fuel Handling Building ventilation system. This amendment is applicable only during the shutdown period prior to Cycle 5 criticality. These tests will be required prior to criticality following authorization to restart.

It was necessary for us to make minor modifications to your request so as to conform with regulatory requirements. Your staff has agreed to these modifications.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Richard H. Jacobs, Project Manager
Operating Reactors Branch #4
Division of Licensing

Enclosures:

1. Amendment No. 76
2. Safety Evaluation
3. Notice

cc w/enclosures: See next page

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DATE	3/30/82	3/30/82	3/31/82	3/31/82	3/31/82		

Metropolitan Edison Company

- 1 -

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- 2 -

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cc w/enclosure(s) & incoming dtd.:
2/25/82

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER AND LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

GPU NUCLEAR CORPORATION

DOCKET NO. 50-289

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 76
License No. DPR-50

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation, et al. (the licensees), dated February 25, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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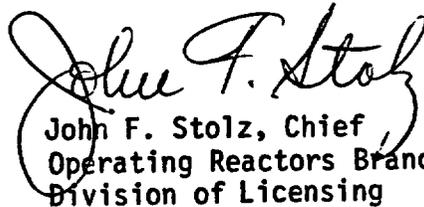
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-50 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 76, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 2, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 76

FACILITY OPERATING LICENSE NO. DPR-50

DOCKET NO. 50-289

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove Pages

3-61

3-62a

3-62c

4-55d

Insert Pages

3-61

3-62a

3-62c

4-55d

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3.15 AIR TREATMENT SYS AS

3.15.1 EMERGENCY CONTROL ROOM AIR TREATMENT SYSTEM

Applicability

Applies to the emergency control room air treatment system and its associated filters.

Objective

To specify minimum availability and efficiency for the emergency control room air treatment system and its associated filters.

Specifications

- 3.15.1.1 Except as specified in Specification 3.15.1.3 below, both emergency treatment systems, AH-E18A fan and associated filter AH-F3A and AH-E188 fan and associated filter AH-F3B shall be operable at all times, per the requirements of Specification 3.15.1.2 below, when containment integrity is required and when irradiated fuel handling operations are in progress.
- 3.15.1.2 a.* The results of the in-place DOP and halogenated hydrocarbon tests at design flows on HEPA filters and charcoal adsorber banks shall show <0.05% DOP penetration and <0.05% halogenated hydrocarbon penetration, except that the DOP test will be conducted with prefilters installed.
- b.* The results of laboratory carbon sample analysis shall show $\geq 90\%$ radioactive methyl iodide decontamination efficiency when tested at 125°F, 95% R.H.
- c. The fans AH-E18A and B shall each be shown to operate within ± 4000 CFM of design flow (40,000 CFM).
- 3.15.1.3 From and after the date that one control room air treatment system is made or found to be inoperable for any reason, reactor operation or irradiated fuel handling operations are permissible only during the succeeding 7 days provided the redundant system is demonstrated to be operable per 4.12.1.1 and 4.12.1.3 within 24 hours and daily thereafter.
- 3.15.1.4 From the date that both control room air treatment systems are made or found to be inoperable or if the inoperable system of 3.15.1.3 cannot be made operable in 7 days, irradiated fuel handling operations shall be terminated in 2 hours and reactor shutdown shall be initiated and the reactor shall be in cold shutdown within 48 hours.

*Not required until criticality for Cycle 5 operation.

3.15.2 REACTOR BUILDING PURGE AIR TREATMENT SYSTEM

Applicability

Applies to the reactor building purge air treatment system and its associated filters.

Objective

To specify minimum availability and efficiency for the reactor building purge air treatment system and its associated filters.

Specification

- 3.15.2.1 Except as specified in Specification 3.15.2.3 below, the Reactor Building Purge Air Treatment System filter AH-F1 and fans AH-E7A and B shall be operable as defined by the Specification below at all times when containment integrity is required.
- 3.15.2.2
- a* The results of the in-place DOP and halogenated hydrocarbon tests at maximum available flows on HEPA filters and charcoal adsorber banks for AH-F1 shall show <0.05% DOP penetration and <0.05% halogenated hydrocarbon penetration, except that the DOP test will be conducted with prefilters installed.
 - b* The results of laboratory carbon sample analysis from the reactor building purge system filter carbon shall show $\geq 90\%$ radioactive methyl iodide decontamination efficiency when tested at 250°F, 95% R.H.
 - c. Fans AH-E7A and B shall each be shown to operate within $\pm 2,500$ CFM of design flow (25,000 CFM).
- 3.15.2.3
- a. From and after the date that the filter AH-F1 in the reactor building purge system is made or found to be inoperable as defined by Specification 3.15.2.2 above, or both fans AH-E7A and B are found to be inoperable, reactor operation is permissible only during the succeeding 30 days, unless such filter and at least one fan is sooner made operable.
 - b. If the required conditions for the reactor building purge filter and fan cannot be met after 30 days; operations shall be terminated immediately and the reactor placed in cold shutdown within 48 hours.

*Not required until criticality for Cycle 5 operation.

Bases

The Reactor Building Purge Exhaust System filter AH-F1 while normally used to filter all reactor building exhaust air, serves also as the post-accident purge filter when used in conjunction with the Hydrogen Purge System to reduce hydrogen gas concentrations in the reactor building following a LOCA. It is necessary to demonstrate operability of these filters to assure readiness for service if required, approximately thirty (30) days following a hypothetical LOCA.

3.15.3 AUXILIARY AND FUEL HANDLING EXHAUST AIR TREATMENT SYSTEM

Applicability

Applies to the auxiliary and fuel handling exhaust air treatment system.

Objective

To specify the minimum availability and efficiency for the auxiliary and fuel handling exhaust air treatment system.

Specification

3.15.3.1 The auxiliary and fuel handling buildings exhaust air treatment system shall be operable at all times when fuel handling operations are in progress in the Fuel Handling Building and whenever irradiated fuel is in the storage pool. This applies to the exhaust filters AH-F2A, 2B, 2C, and 2D as well as the exhaust fans AH-E14A, 14B, 14C, and 14D.

From and after the date that the auxiliary and fuel handling exhaust air treatment system is made or found to be inoperable, that is the filters AH-F 2A, 2B, 2C, and 2D and/or both sets of fans AH-E 14A and 14C and AH-E 14B and 14D, are inoperable, fuel handling operations shall be terminated immediately until the components are returned to service. Any fuel assembly movement in progress may be completed.

- 3.15.3.2* a. The results of the in-place DOP and halogenated hydrocarbon tests at design flows on HEPA filters and charcoal adsorber banks shall show <0.05% DOP penetration and <0.05% halogenated hydrocarbon penetration, except that the DOP test will be conducted with prefilters installed.
- b. The results of laboratory carbon sample analysis shall show $\geq 90\%$ radioactive methyl iodide decontamination efficiency when tested at 125°F, 95% R.H.
- c. Each set of fans AH-E14 A & C and AH-E14 B & D shall each be shown to have the capacity of operating within $\pm 11,881$ CFM of design flow (118,870 CFM).

3.15.3.3 With one auxiliary and fuel handling exhaust air treatment system inoperable, fuel movement within the storage pool may proceed provided the OPERABLE auxiliary and fuel handling exhaust air treatment system is in operation and discharging through at least one train of HEPA filters and charcoal adsorbers.

*Not required until criticality for Cycle 5 operation.

4.12.3 AUXILIARY AND FUEL HANDLING EXHAUST AIR TREATMENT SYSTEM

Applicability

Applies to the auxiliary and fuel handling building exhaust air treatment system and associated components.

Objective

To verify that this system and associated components will be able to perform its design functions.

Specification

- 4.12.3.1 At least once per refueling interval or once per 18 months, whichever comes first, it shall be demonstrated that the pressure drop across the combined HEPA filter and adsorber banks is less than 6 inches of water at system design flow rate (+10%).
- 4.12.3.2 a.* The tests and sample analysis required by Specification 3.15.3.2 shall be performed initially, once per refueling interval or 18 months, whichever comes first, or within 30 days prior to the movement of irradiated fuel and following significant painting, steam, fire, or chemical release in any ventilation zone communicating with the system that could contaminate the HEPA filters or charcoal adsorbers.
- b. DOP testing shall be performed after each complete or partial replacement of a HEPA filter bank or after any structural maintenance on the system housing that could affect the HEPA filter bank bypass leakage.
- c. Halogenated hydrocarbon testing shall be performed after each complete or partial replacement of a charcoal adsorber bank or after any structural maintenance on the AH-F 2A, B, C, or D housing that could affect charcoal adsorber bank bypass leakage.
- d. The fan combination AH-E 14A and C and AH-E 14B and D shall be operated at least 10 hours every month.
- 4.12.3.3* An air distribution test shall be performed on the HEPA filter bank initially and after any maintenance or testing that could affect the air distribution within the system. The air distribution across the HEPA filter bank shall be uniform within +20%. The test shall be performed at 118,810 cfm (+10% flow rate).

Bases

Pressure drop across the combined HEPA filters and charcoal adsorbers of less than 6 inches of water at the system design flow rate will indicate that the filters and adsorbers are not clogged by excessive amounts of foreign matter. Pressure drop should be determined at least once every refueling interval to show system performance capability.

*Surveillance to be performed prior to Cycle 5 criticality in lieu of other criteria specified. This change is only applicable until Cycle 5 criticality.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 76 TO FACILITY OPERATING LICENSE NO. DPR-50

METROPOLITAN EDISON COMPANY
JERSEY CENTRAL POWER AND LIGHT COMPANY
PENNSYLVANIA ELECTRIC COMPANY
GPU NUCLEAR CORPORATION

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-289

Introduction

By letter dated February 25, 1982 (TSCR 111, 82-031), GPU Nuclear Corporation (the licensee) requested an amendment to Appendix A of Facility Operating License No. DPR-50 for the Three Mile Island Nuclear Station, Unit No. 1 (TMI-1). This amendment would permit fuel movements at TMI-1 without requiring the tests to verify the efficiency of the air filter systems for the Reactor Building purge exhaust system, the Control Building emergency ventilation system and the Auxiliary and Fuel Handling Building ventilation system. In addition, this amendment would permit a relaxation of ventilation system flow rate requirements for the Auxiliary and Fuel Handling Building ventilation system. This amendment is applicable only for the shutdown period prior to Cycle 5 criticality. The licensee has indicated that these changes to the Technical Specifications (TSs) are needed to permit a timely investigation of possible degradation of materials within the reactor vessel.

On March 12, 1982, the licensee filed a motion before the Atomic Safety and Licensing Board in the TMI-1 Restart proceeding seeking clarification of a Board ruling on the need for an engineered safety features filter system for the Fuel Handling Building. Attached to this motion is an affidavit by Robert W. Keaten of GPU Nuclear Corporation, which provides the results of an analysis of a fuel handling accident in the Fuel Handling Building. We considered this affidavit in our evaluation of the requested amendment.

Discussion and Evaluation

The purpose of the TSs which the licensee seeks to amend is to provide assurance that the air filter systems for the Reactor Building purge exhaust, Control Room emergency ventilation system and the Auxiliary and Fuel Handling Building ventilation system are capable of performing their intended function. The function of these systems is to provide for the treatment and controlled release of radioactive materials in the event of an accident which has the potential to release radioactive material. When the reactor is shutdown, the accidents of concern involve movements of irradiated fuel. TMI-1 has been shutdown for more than three years.

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The air filter systems consist of High Efficiency Particulate Air (HEPA) filters and other filters, charcoal adsorber beds and fans. The charcoal adsorbers function to remove airborne radioiodines. The HEPA and other filters are provided to remove other airborne particulates which could impair the efficiency of the charcoal adsorbers. The fans serve to control the overall flow of air in the affected space and control the flow rate of air through the charcoal adsorbers so as not to impair their function. The licensee has proposed changes to the TSs which would allow fuel movements at Unit 1 without conducting tests to verify the efficiency of the HEPA filters and charcoal adsorber beds of the air filter systems in the Reactor Building purge exhaust system, the Control Room emergency ventilation system and the Auxiliary and Fuel Handling Building ventilation system. In addition, the licensee seeks relaxation of flow rate requirements for the fans in the Auxiliary and Fuel Handling Building. Relief from these TS requirements applies only to the shutdown period prior to the next (Cycle 5) criticality.

Current licensing guidelines for evaluating the system design features for mitigating the consequences of fuel handling accidents are set forth in Section 15.7.4 of the Standard Review Plan (SRP). That section of the SRP indicates that systems for mitigating the radiological consequences of a fuel handling accident are acceptable if the calculated whole-body and thyroid dose at the exclusion area boundary and low population zone (LPZ) boundary are well within the exposure guideline values of 10 CFR Part 100, Section 100.11. "Well within" means 25% or less of the 10 CFR Part 100 exposure guideline values (75 rem for the thyroid and 6.25 rem for the whole-body doses are 25% of the 10 CFR Part 100 values).

In order to assess the potential hazard posed by a fuel handling accident at TMI-1 prior to restart, the licensee has calculated the results of a postulated fuel handling accident without taking credit for any filtering of the discharge from the Fuel Handling Building (Robert W. Keaten affidavit). For the purposes of the calculation, it was assumed that TMI-1 had been shutdown and that fuel activity has decayed for a period of two-and-a-half years. All other assumptions and methodology used in the analysis were in accordance with the guidelines of SRP 15.7.4 and Regulatory Guide 1.25.

Regulatory Guide 1.25 provides acceptable assumptions that may be used for evaluating the potential radiological consequences of a fuel handling accident in the fuel handling and storage facility for boiling and pressurized water reactors.

The results of the licensee's dose calculation are as follows:

	<u>Exclusion Boundary</u>	<u>LPZ</u>	<u>10 CFR 100 Limit</u>	<u>25% of Limit</u>
Thyroid dose (rem)	8.7×10^{-5}	1.7×10^{-5}	300	75
Whole-body dose (rem)	6.5×10^{-4}	1.3×10^{-4}	25	6.25

The calculated doses are very low due mainly to the extensive time period since the TMI-1 fuel assemblies were last irradiated. Since the reactor is to remain shutdown, virtually no radioiodine exists which could potentially be released to the atmosphere in the event of a fuel handling accident, even in the absence of a filter system to control the release of radioiodines. Although we have not performed independent calculations to verify the above values, the licensee's values are considered reasonable for the condition of the Unit 1 fuel, and in any case, the actual doses at the exclusion boundary and LPZ would be orders of magnitude below the acceptable limits.

The above calculations demonstrate that in the event of a fuel handling accident with the Unit 1 fuel in its present condition, the offsite doses would be only a very small fraction of acceptable limits, even if there were no filter systems to treat the release. If offsite doses are acceptable with no filter system in use, it follows that tests to demonstrate the efficiency of the filter systems are not important for this fuel movement.

In addition, since no additional radioiodines in Unit 1 fuel will be generated until criticality, we agree with the licensee that these tests are not needed until Cycle 5 criticality. For the above reasons, we find the licensee's request to amend the TSs to defer until Cycle 5 criticality testing of the HEPA filters and charcoal adsorbers of the filter systems of the Reactor Building purge exhaust system, the Control Room emergency ventilation system and the Auxiliary and Fuel Handling Building ventilation system to be acceptable.

With respect to the licensee's request for relief from ventilation system flow rate requirements for the Auxiliary and Fuel Handling Building, the licensee has indicated that the system was capable of developing approximately 103,000 cfm during tests instead of the TS flow rate of 118,810 cfm \pm 10%. As with the rest of this amendment, this request applies only to the shutdown period prior to Cycle 5 criticality. Since the intent of requiring specific flow rates relates to the efficiency of the charcoal adsorber to remove airborne radioiodines and since virtually no iodines will exist until plant operation, we conclude that requiring specific flow rate requirements is unnecessary for these fuel movements. We note, however, that additional justification will be necessary to show that the reduced flow rate is sufficient in the event of accidents following Cycle 5 criticality.

For the above reasons, we find the proposed amendment acceptable.

¹We note that this analysis would not be applicable to the Control Room emergency ventilation system, but since virtually no radioiodine exists, we similarly have no concern that the efficiency of this filter system requires verification prior to these fuel movements.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

The following NRC staff personnel have contributed to this Safety Evaluation: C. Nichols, R. Jacobs.

Dated: April 2, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-289METROPOLITAN EDISON COMPANYJERSEY CENTRAL POWER AND LIGHT COMPANYPENNSYLVANIA ELECTRIC COMPANYGPU NUCLEAR CORPORATIONNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 76 to Facility Operating License No. DPR-50, issued to Metropolitan Edison Company, Jersey Central Power and Light Company, Pennsylvania Electric Company, and GPU Nuclear Corporation (the licensees), which revised the Technical Specifications (TSs) for operation of the Three Mile Island Nuclear Station, Unit No. 1 (the facility) located in Dauphin County, Pennsylvania. The amendment is effective as of its date of issuance.

The amendment revises the TSs for the facility to defer until Cycle 5 criticality, tests to verify the efficiency of HEPA filters and charcoal adsorbers in the filter systems for the Reactor Building purge exhaust, the Control Room emergency ventilation system, and the Auxiliary and Fuel Handling Building ventilation system. This amendment is applicable only during the shutdown period prior to Cycle 5 criticality. These tests will be required prior to criticality following authorization to restart.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and

regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated February 25, 1982, (2) Amendment No.76 to License No. DPR-50, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. 20555, and at the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania 17126. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 2nd day of April 1982,

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing