



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER AND LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

DOCKET NO. 50-289

THREE MILE ISLAND NUCLEAR STATION, UNIT 1

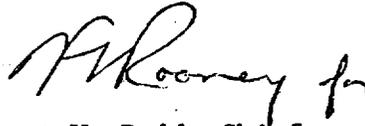
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 16
License No. DPR-50

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Metropolitan Edison Company, Jersey Central Power and Light Company, and Pennsylvania Electric Company (the licensees) dated April 15, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. After weighing the environmental aspects involved, the issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Dated: ~~MAY~~ 12 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 16

FACILITY OPERATING LICENSE NO. DPR-50

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Remove pages 4-33 and 4-34 from the Appendix A Technical Specifications and insert the attached replacement pages. The changed areas on the revised page are shown by marginal lines. Page 4-34 is unchanged and is included for convenience only.

c. The following isolation valves will be tested by either testing the Fluid Block System or by measuring the local leak rate for all of these valves using a type "C" test as defined by 10 CFR 50, Appendix J.

1. Nuclear Service Closed Cooling Water (NS-V4 and NS-V15)
2. Intermediate Cooling Water (IC-V3, V4 and V6)
3. Spent Fuel Cooling (SF-V23)
4. Make-up and Purification (MU-V3 and MU-V26)
5. Reclaimed Water (CA-V189)
6. Sample Valves (CA-V5A&B and CA-V2)
7. Drain Valves (WDL-V304, WDG-V4 and WDL-V535)

d. The following isolation valves or blank flanges will be tested by testing the Penetration Pressurization System:

1. Instrument Air (IA-V6 and IA-V20)
2. Service Air (SA-V2 and SA-V3)
3. Leak rate testing (LR-V1, 2, 3, 4, 5, 6 and 10)
Blank flanges on Penetrations 414, 415, 416
4. Incore Inst. Transfer Tube - Blank flange on Penetration 241

4.4.1.2.2 Conduct of Tests

- a. Local leak rate tests shall be performed pneumatically at a pressure of not less than P_a .
- b. Acceptable methods of testing are halogen gas detection, pressure decay, pneumatic flow measurement or equivalent.
- c. The pressure for a valve test shall be applied in the same direction as that when the valve would be required to perform its safety function unless it can be determined that the direction will provide equivalent or more conservative results.
- d. Valves to be tested shall be closed by normal operation and without any preliminary exercising or adjustments.

4.4.1.2.3 Acceptance Criteria

The combined leakage from all items listed in 4.4.1.2.1, except leakage from those valves or devices sealed by the Fluid Block System* or Penetration Pressurization System, shall not exceed .6 L_a (the maximum allowable leakage rate at P_a).

*When type "C" testing is used for the valves served by the Fluid Block System, their air/nitrogen leakage shall be included in the combined leakage.

4.4.1.2.4 Corrective Action and Retest

- a. If at any time it is determined that the criterion of 4.4.1.2.3 above is exceeded, repairs shall be initiated immediately.
- b. If conformance to the criterion of 4.4.1.2.3 is not demonstrated within 48 hours following detection of excessive local leakage, the reactor shall be shutdown and depressurized until repairs are effected and the local leakage meets the acceptance criterion as demonstrated by retest.

4.4.1.2.5 Test Frequency

Local leak detection tests shall be performed at a frequency of at least each refueling period, except that:

- a. The equipment hatch and fuel transfer tube seals shall be tested every other refueling period but in no case at intervals greater than 3 years. If they are opened they will be tested after being closed.
- b. The resilient seal of the personnel and emergency air locks' outer doors shall be tested at six month intervals, except when the air locks are opened during that interval. In this case they shall be tested after each use but no more than once daily. During Cold S/D the minimum test frequency is one per week.
- c. The reactor building purge isolation valves shall be tested yearly.
- d. Readings of the rotameters in each manifold of the penetration pressurization system shall be recorded at periodic intervals not to exceed three months.

4.4.1.3 Isolation Valve Functional Tests

Every three months, remotely operated reactor building isolation valves shall be stroked to the position required to fulfill their safety function unless such operation is not practical during plant operation. The valves not stroked every three months shall be stroked during each refueling period.

4.4.1.4 Annual Inspection

A visual examination of the accessible interior and exterior surfaces of the containment structure and its components shall be performed annually and prior to any integrated leak test to uncover any evidence of deterioration which may affect either the containment's structural integrity or leak-tightness. The discovery of any significant deterioration shall be accompanied by corrective actions in accord with acceptable procedures, nondestructive tests, and inspections, and local testing where practical, prior to the conduct of any integrated leak test. Such repairs shall be reported as part of the test results.