



UNITED STATES
ATOMIC ENERGY COMMISSION
WASHINGTON, D.C. 20545

METROPOLITAN EDISON COMPANY
JERSEY CENTRAL POWER AND LIGHT COMPANY
PENNSYLVANIA ELECTRIC COMPANY

DOCKET NO. 50-289

(Three Mile Island Nuclear Station, Unit 1)

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 4
License No. DPR-50

1. The Atomic Energy Commission (the Commission) having found that:
 - A. The application for amendment by Metropolitan Edison Company, Jersey Central Power and Light Company and Pennsylvania Electric Company (the licensees) dated July 17, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended, and the Commission's rules and regulations;
 - B. The facility will operate in conformity with the license, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.
2. Accordingly, Paragraph 2.C.(2) of Facility License No. DPR-50 is hereby amended to read as follows:

"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B attached to Facility Operating License No. DPR-50 are revised as indicated in the attachment to this license amendment. The Technical Specifications, as revised, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications, as revised."

3. This license amendment is effective as of July 12, 1974.

FOR THE ATOMIC ENERGY COMMISSION

Original signed by
Voss A. Moore

Voss A. Moore, Assistant Director
for Light Water Reactors, Group 2
Directorate of Licensing

Attachment:
Change No. 3 to Appendix A
Technical Specifications
License No. DPR-50

Date of Issuance: AUG 16 1974

TURBINE CYCLE

Applicability

Applies to the operating status of the turbine cycle when fuel is in the core and the capability of the decay removal system is exceeded.

Objective

To specify minimum conditions of the turbine cycle steam relieving capacity and emergency feedwater pumps to ensure capability to remove decay heat equivalent to five percent of full power.

Specification

The reactor coolant system shall not be heated above 250°F with irradiated fuel in the core unless the following conditions are met:

- 3.4.1 Four of the six turbine bypass valves are operable.
- 3.4.2 The turbine driven emergency feedwater pump and two half-sized motor-driven emergency feedwater pumps shall be operable.
- 3.4.3 A minimum of 150,000 gallons of condensate is available in each condensate storage tank.
- 3.4.4 System piping and valves, directly associated with the above components are operable.
- 3.4.5 Any component cited above may be removed from service for purposes of maintenance for a period not to exceed 48 hours. If this period is exceeded the unit will be placed in the cold shutdown condition within 12 hours thereafter.
- 3.4.6 All eighteen main steam safety valves shall be operable or, if any are not operable, the maximum overpower trip setpoint (see Table 2.3-1) shall be reset as follows:

<u>Maximum Number of Safety Valves Disabled On Any Steam Generator</u>	<u>Maximum Overpower Trip Setpoint (% of Rated Power)</u>
1	92.4
2	79.4
3	66.3
*	

*Operation with more than three safety valves disabled on any steam generator is not permitted.

Bases

A reactor shutdown from power requires removal of core decay heat. Normal decay heat removal is by the steam generator with the steam dump to the condenser when system temperature is above 250°F and by the decay heat removal system below 250°F. Therefore, core decay heat can be continuously dissipated up to 15 percent of full power via the steam bypass to the condenser as feedwater in the steam generator is converted to steam by heat absorption. Normally, the capability to return feedwater flow to the steam generators is provided by the main feedwater system.