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ROBERT C. MECREDY **Vice President Nuclear Operations**

October 18, 2000

U.S. Nuclear Regulatory Commission Document Control Desk Attn: Guy S. Vissing Project Directorate I Washington, D.C. 20555

LER 2000-001, Intermediate Range Channel Loss of Control Power, Due to Subject: **Excessive Signal Noise, Results in Reactor Trip R.E.** Ginna Nuclear Power Plant Docket No. 50-244

Dear Mr. Vissing:

The attached Licensee Event Report LER 2000-001, is submitted in accordance with 10 CFR 50.73, Licensee Event Report System, item (a) (2) (iv), which requires a report of, "Any event or condition that resulted in a manual or automatic actuation of any engineered safety feature (ESF), including the reactor protection system (RPS)".

Very truly yours,

Robert C. Mecredy

Mr. Guy S. Vissing (Mail Stop 8C2) xc: Project Directorate I **Division of Licensing Project Management** Office of Nuclear Reactor Regulation **U.S. Nuclear Regulatory Commission** Washington, D.C. 20555

> Regional Administrator, Region I **U.S. Nuclear Regulatory Commission** 475 Allendale Road King of Prussia, PA 19406

U.S. NRC Ginna Senior Resident Inspector

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<u> </u>																
NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISS (6-1998)				MISSI	ON AI	PRC timate	VED BY OMB NO. of burden per response	3150-0104 to comply with	EXP this man	IRES 06/30/2001 datory information						
(6-1998)		LICI	ENSEE EV (See reverse digits/char	VENT REPO for required nu acters for each	DRT (LI umber of block)	ER)			col the bu Nu Pa Bu a c an	collection request: 50 hrs. Reported lessions learned are incorporated in the licensing process and fed back to industry. Forward comments regardin burden estimate to the Records Management Branch (T-6 F33), U.S Nuclear Regulatory Commission, Washington, DC 20555-0001, and to th Paperwork Reduction Project (3150-0104), Office of Management ar Budget, Washington, DC 20503. If an information collection does not displa a currently valid OMB control number, the NRC may not conduct or sponse and a person is not required to respond to, the information collection.						
FACILITY	NAME (1)	<u></u>							CKE	T NUMBER (2)			PAGE (3)		
REG	inna N	luclear	Power Pla	ent						0!	5000244		1	OF 8		
TITLE (4)												<u>n</u>				
Interme	diate l	Range	Channel L	oss of Contr	ol Power	, Due t	o Ex	xcess	ive Sig	nal	Noise, Results ir	Reactor 7	Fri p			
EVEN		E (5)		ER NUMBER (6)	RE	POF	T DA	NE (7)		OTHER F	ACILITIES IN	VOLVE	D (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION	MON	тн	DAY	YEAR	FA	CILITY NAME		DOCKET 0500	NUMBER DO		
09	18	2000	2000	- 001 -	00	10	,	18	2000	FA	CILITY NAME		DOCKET	NUMBER DO		
			ТНІ	S REPORT IS		D PURS	UAN	IT TO	THE RE	QUIF	EMENTS OF 10 CF	R§: (Check	one or	more) (11)		
MODE	E (9)	3	20.220)1(b)		20.22	203(a	a)(2)(v)			50.73(a)(2)(i)		50.7	73(a)(2)(viii)		
POWER LEVEL (10) 000		20.2203(a)(1)			20,22	203(2	(a)(3)(i)			50.73(a)(2)(ii)		50.	73(a)(2)(x)			
LEVEL	. (10)	000	20.220)3(a)(2)(i)		20.22	203(a	a)(3)(ii)			50.73(a)(2)(iii)		73.	71		
			20.220)3(a)(2)(ii)		20.22	203(a NoV4	<u>3)(4)</u> 1)		 x	50.73(a)(2)(iv)			HER		
20.2203(a 20.2203(a)3(a)(2)(iv)	50.36(c)(2)				50.73(a)(2)(vii)			or in NRC Form 366A					
					LICENS	EE CON	TAC	T FOR	THIS L	ER (1	12)					
NAME	St I	Martin	- Technic	al Assistant						TE	ELEPHONE NUMBER (Incl (7	ude Area Code) 16) 771-3	641			
						COMBO				ESC						
CAUSE	eve	TEM	COMPONENT			EPORTAB	LE		USE	SYST		MANUFACT	URER	REPORTABLE		
CAUSE																
В	<u> </u>	G		W120	<u> </u>	۲ 										
		SU	PPLEMENT/	L REPORT E	PECTED	(14)					EXPECTED	MONTH	DAY	YEAR		
YES (If yes	s, comple	ete EXPE	CTED SUBMIS	SION DATE).			x	NO			DATE (15)					
ABSTRA	CT (Lir	nit to 140	0 spaces, i.e., a	approximately 15	single-space	d typewritt	ten lin	ies) (1	5)							
On Ser	ntemh	er 18	2000 st	approximat	elv 0503	B EDS'	T. fl	he nl	ant wa	s in	Mode 3 with t	he reactor	coola	int system		
being	maint	ained	at a tempe	erature betw	veen 540) degre	es l	F and	547 d	legr	ees F and a pre	ssurizer p	ressur	re of		
approx	timate	ly 223	35 psig. I	A planned p	lant shu	tdown	wa	s in ŗ	orogres	ss, i	n preparation f	or beginni	ing the	e 2000		
refueli causin	ng ou g a re	itage. actor 1	During th rip.	is shutdown	n, a fuse	blew	ın a	nuc	ear in	stru	ment system in	termediat	e rang	e circuit,		
The C	ontrol	Roon	n operator	s performe	d the app	propria	ate a	action	ns of p	roc	edures E-0 and	ES-0.1 .	Follov	ving the		
reactor	r trip,	all sy	stems ope	rated as des	signea.		-	•		~		ς.				
Immed	liate o	correct	tive action	n was taken	to stabi	lize the	e pl	ant ir	n Mod	e 3.						

The cause of the blown fuse was excessive signal noise in the intermediate range circuit.

Corrective action to prevent recurrence is outlined in Section V.B.

NRC FORM 366A (6-1998)				J.S. NUCLEAR RE	GULATORY	COMMIS	SION
	LICENSEE EVEN TEXT CON					3	·····
FAC	CILITY NAME (1)	DOCKET (2) NUMBER (2)	5)	PAGI	E (3)		
R. E. Ginna Nuclear Pow	C FORM 366A (1996) ILICENSEE EV TEXT C FACILITY NAME (1) . E. Ginna Nuclear Power Plant EXT (If more space is required, use additional copies of NRC Form 36 PRE-EVENT PLANT CONDITIONS: On September 18, 2000, a planned plant shutd O-2.1, "Normal Shutdown to Hot Shutdown" The plant was subcritical in Mode 3 and contreactor shutdown for the 2000 refueling outag completely inserted (to zero steps), and Bank (RCS) was being maintained at a temperature pressurizer pressure of approximately 2235 p documented in the Official Record, the reactor Reactor power was very low in the intermedia amount of negative reactivity being added by At approximately 0502 EDST, Bank "A" rod approximately 70 steps. Reactor power, as in range (IR) channels, was approaching the per is automatically reset from NIS IR channels v of P-6 automatically re-energizes the NIS sou DESCRIPTION OF EVENT: A. DATES AND APPROXIMATE TIM September 18, 2000, 0503 ED September 18, 2	05000244	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF	8
			2000	- 001 -	00		
TEXT (If more space is require I. PRE-EVENT P	d, use additional copies of NRC Form 366A) LANT CONDITIONS:	(17)					
O-2.1, "Normal The plant was s reactor shutdow completely inse (RCS) was bein pressurizer press documented in Reactor power amount of nega At approximately range (IR) chan is automatically of P-6 automatic	Shutdown to Hot Shutdown", in subcritical in Mode 3 and control is on for the 2000 refueling outage. Erted (to zero steps), and Bank "A ing maintained at a temperature be soure of approximately 2235 psig. the Official Record, the reactor we was very low in the intermediate strive reactivity being added by con- ely 0502 EDST, Bank "A" rods ha 70 steps. Reactor power, as indic- innels, was approaching the permiss of reset from NIS IR channels whe ically re-energizes the NIS source	preparation f rods were bei Rods for Con "rods were bei tween 540 de Although of vas, in fact, su range and wa ntrol rod inser- ad been insert ated on Nucl ssive P-6 rese range (SR) of	for begin ing inser- ntrol Bar- being dri grees F fficially ubstantia is contin- rtion. ted from ear Instr- t setpoin hannels channels	ming the 2000 ted into the conks "D", "C", ven in. The read and 547 degree in Mode 2 as a lly subcritical ually decreasing the full out per ument System at of 5E-11 and decrease below	o refueling ore to com and "B" H eactor coo ees F and a conservat and in M ng due to osition to a (NIS) in aps. Perm w 5E-11 a	g outage plete th had been blant sys a ively ode 3. the larg termedi hissive H umps. R	e. ne stem ge ate 2-6 Reset
II. DESCRIPTION	N OF EVENT:		,	ł			
A. DATES	AND APPROXIMATE TIMES	OF MAJOR	OCCUI	RRENCES:			
•	September 18, 2000, 0503 EDST	: Event date a	and time	•			
•	September 18, 2000, 0503 EDST	: Discovery d	late and	time.			
•	September 18, 2000, 0503 EDST breakers open and verify all contr	: Control Roo rol and shutd	om opera own rod	ators verify bo s inserted.	oth reactor	r trip	
•	September 18, 2000, 0507 EDST steam isolation valves to limit a r	: Control Ro eactor coolar	om oper nt systen	ators manually 1 cooldown.	y close bo	th main	l
•	September 18, 2000, 0531 EDST	: Plant is stat	oilized ir	n Mode 3.			

NRC FORM 366A (6-1998)

NRC FORM 36F	δA		ļ	U.S. N	UCLEAR RE	EGULATORY	COMMIS	SION			
-1990)	LICENSEE EVEN TEXT CON		(LER)								
<u></u>	FACILITY NAME (1)	DOCKET (2) NUMBER (2)		LEF	R NUMBER ((6)	PAG	;E (3			
		05000044	YEAR SEQUENTIAL REVIS	YEAR SEQUENTIAL REV NUMBER NUM	SEQUENTIAL REVISIONNUMBER NUMBER						
ł. E. Ginna r	Nuclear Power Plant	05000244	2000	-	001 -	- 00	3 OF	:			
EXT (If more s	space is required, use additional copies of NRC Form 366A)	(17)									
B.	EVENT:										
	intermediate range (IR) channels, was approaching the permissive P-6 reset setpoint of 5E-11 amps. Permissive P-6 is automatically reset from NIS IR channels when 2 of 2 IR channels decrease below 5E-11 amps. Reset of P-6 automatically re-energizes the NIS source range (SR channels.										
	The P-6 reset setpoint was reached at approximately 0503 EDST. At the same time as the char of state of the P-6 bistable for NIS IR channel N-36, a control power fuse blew in channel N-36 resulting in loss of control power to the N-36 channel. This loss of power de-energized the NI IR high flux trip reactor trip relay for channel N-36 and the reactor tripped on 1 of 2 NIS IR high flux range trip. In addition to numerous Main Control Board (MCB) annunciators already in alarm from the ongoing plant shutdown, the Control Room operators acknowledged MCB annunciator D-18 "Intermediate Range Reactor Trip $1/2 25$ %", indicating a reactor trip from NIS IR channel N-36.										
	The two reactor trip breakers opened as designed and all shutdown and control rods that were withdrawn inserted as designed.										
	The Control Room operators performed to Procedure E-0, "Reactor Trip or Safety In Procedure ES-0.1, "Reactor Trip Respon were open, all control and shutdown rode required.	the appropriat njection". Th se", when it v s were inserte	te action ey trans was veri d, and s	ns of sition fied t afety	Emergen led to Em that both injection	icy Operationergency C reactor trip n was not a	ing peratin p break actuate	ıg :er d (
	During the performance of ES-0.1, stean system (RCS) cooldown. Both main ste	n generator bl am isolation v	lowdowr valves (1	n flov MSIV	w was car Vs) were :	using a rea manually (ictor co closed)o] to			

limit the RCS cooldown.

The plant was stabilized in Mode 3 at approximately 0531 EST and the Control Room operators transitioned back to normal plant operating procedure O-2.1.

NRC FORM 366	A			J.S. N	UCLEAR	REGU	JLATORY	COMMIS	SION	
(0-1998)	LICENSEE EVEN TEXT CON	IT REPORT	(LER)							
	FACILITY NAME (1)	DOCKET (2) NUMBER (2)		LER	NUMBE	R (6)		PAG	E (3)	
			YEAR	SE	EQUENTIAL NUMBER	- '	REVISION NUMBER			
R. E. Ginna N	IC FORM 366A 1988) LICENSEE EVE TEXT CC FACILITY NAME (1) E. Ginna Nuclear Power Plant The reactor trip (referred to as "scram" reactor was subcritical. Thus, this scra Indicator (PI) "Unplanned Scrams Per definition for the NRC PI "Scrams Wit subcritical prior to the scram and the ne 2) were removed due to intentional ope C. INOPERABLE STRUCTURES, COM THE EVENT: None D. OTHER SYSTEMS OR SECONDAR None E. METHOD OF DISCOVERY: This event was immediately apparent of due to plant response and alarms and it F. OPERATOR ACTION: After the reactor trip, the Control Rood Emergency Operating Procedures E-0 cooldown, and the plant was stabilized Subsequently, the Control Room opera 10 CFR 50.72 (b) (2) (ii), non-emerger September 18, 2000.	05000244	2000	-	001		00	4 OF	8	
TEXT (If more s	pace is required, use additional copies of NRC Form 366A)	(17)					-			
	The reactor trip (referred to as "scram" in reactor was subcritical. Thus, this scram Indicator (PI) "Unplanned Scrams Per 7, definition for the NRC PI "Scrams With subcritical prior to the scram and the norn 2) were removed due to intentional opera	NRC Perfor did not meet 000 Critical H a Loss of Non nal heat remo tor actions.	mance I the defi Iours". rmal Hea oval path	ndica nitio The at Re ns (as	ators) o n for th scram a moval" s listed	ccuri e NF ilso c ' sinc in N	red whe C Perfe did not 1 ce the re EI 99-0	n the ormanc meet th actor v 2,Revis	e e vas sion	
C.	INOPERABLE STRUCTURES, COMPO THE EVENT:	ONENTS, OI	R SYST	EMS	THAT	CO	NTRIB	UTED	то	
	None									
D.	OTHER SYSTEMS OR SECONDARY	FUNCTION	S AFFE	CTE	D:					
	None	EMS OR SECONDARY FUNCTIONS AFFECTED DISCOVERY: immediately apparent due to Main Control Board incorponse and alarms and indications in the Control Room								
E.	METHOD OF DISCOVERY:									
	This event was immediately apparent due due to plant response and alarms and ind	e to Main Con ications in the	ntrol Bo e Contro	ard i ol Ro	ndicatio om.	on of	f the rea	ctor trij	ρ,	
F.	OPERATOR ACTION:									
	After the reactor trip, the Control Room operators performed the appropriate actions of Emergency Operating Procedures E-0 and ES-0.1. The MSIVs were closed to limit a RCS cooldown, and the plant was stabilized in Mode 3.									
	Subsequently, the Control Room operato 10 CFR 50.72 (b) (2) (ii), non-emergency September 18, 2000.	rs notified hi y four hour n	gher sur otificatio	oervi: on, at	sion and t approx	d the kima	NRC p ntely 064	er 17 EDS	T on	
	(c. 4000)									
NRU FURM 300A (u-1990)									

NRC F (6-1998)	ORM 366.)	A LICENSEE EVEN	T REPORT	י (LER)	J.S. NUCLEAR RE	GULATORY	COMMIS	SION
		TEXT CON	TINUATION				n 	
		FACILITY NAME (1)	DOCKET (2) NUMBER (2)		LER NUMBER (6	5)	PAGE	E (3)
RF	Ginna N	luclear Power Plant	05000244	YEAR	YEAR SEQUENTIAL REVISI NUMBER NUMB		ATORY COMMISS PAGE VISION 5 OO 5 OF Pailure. However oo 5 OF Pailure. However oo 5 OF Page Solution Solution Vision 5 OF Page Solution Solution Page Solution Solution Page Solution Solution oo 5 OF OF Page Solution Solution Solution Solution Solution <thsolution< th=""> Solution <th></th></thsolution<>	
	L. E. Ginna Nuclear Fower Flanc		U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT (LER) TEXT CONTINUATION DOCKET (2) LER NUMBER (6) PAGE (3) VEAR SEQUENTIAL NUMBER (2) VEAR SEQUENTIAL INVIDENTIAL INVIDENTIAL NUMBER (2) VEAR SEQUENTIAL INVIDENTIAL INVIDENTIAL DOCKET (2) VEAR NUMBER (2) VEAR SEQUENTIAL INVIDENTIAL INVIDENTIA					
TEXT ((If more s	pace is required, use additional copies of NRC Form 366A)	(17)					
	G.	SAFETY SYSTEM RESPONSES:						
		None. For Maintenance Rule purposes, to it does not meet the definition for the NR condition that alone could have prevented systems ". The NIS IR high flux trip and is not credited in any safety analysis. reactor protection system going to its fail	his event is cl C PI "Safety I the fulfillme is a backup to In addition, -safe (de-ener	assified System ent of the the NI the loss gized) o	as a Function Functional Fai e safety function S power range of control power condition and t	al Failure ilure": "ar on of stru low rang ver resulte ripping th	. Howe ny even ctures o e flux tu ed in the ne reacte	ver, t or r rip e or.
III.	CAU	ISE OF EVENT:			i -		,	
	А.	IMMEDIATE CAUSE:						
		The immediate cause of the reactor trip w trip logic for NIS IR high flux trip on cha	vas achieving nnel N-36.	the 1 of	2 reactor prot	ection sys	stem (R	PS)
	B.	INTERMEDIATE CAUSE:						
		The intermediate cause of achieving 1 of for NIS IR channel N-36 high flux, due to	2 RPS trip lo o a blown con	gic was trol pov	de-energizing ver fuse.	the react	or trip r	elay
	C. ⁴	ROOT CAUSE:	• •					
		The underlying cause of blowing of the c noise in the N-36 detector circuit. This n the logarithmic current amplifier (log cur caused the current signal (nuclear flux sig fluctuate. When the permissive P-6 setpo bistable excessively. This changed the st and de-energizing the P-6 relay, which re	control power toise was attri rent amplifier gnal) output fi pint was appro- tate of the P-6 esulted in blow	fuse for buted to r) in the rom the bached, bistable wing the	channel N-36 high AC ripp N-36 drawer. faulted log cu this cycled the e, rapidly and control power	was exce le on the This AC rrent amp permissi repeatedly r fuse.	essive si output c ripple lifier to we P-6 y energi	gnal)f
		This event is NUREG-1022 Cause Code	(B), "Design,	, Manufa	acturing, Cons	truction/I	nstallati	ion".
NRC FO	RM 366A (6	3-1998)	<u></u>		<u></u>	<u> </u>	PAGE (5 OF Howev y event of tures or e flux trig d in the e reactor tem (RP) or trip rel ssive sig putput of ripple lifier to ve P-6 y energiz	

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NRC F0 (6-1998)	DRM 366A		U	I.S. NUCLEAR R	EGULATORY	COMMISSION
		IT REPORT	(LER)			
	FACILITY NAME (1)	DOCKET (2)	[LER NUMBER	(6)	PAGE (3)
		_ KOMBER (2)	YEAR	SEQUENTIAL NUMBER	REVISION	
R. E. (LICENSEE E TEXT FACILITY NAME (1) E. Ginna Nuclear Power Plant XT (If more space is required, use additional copies of NRC Form ANALYSIS OF EVENT: This event is reportable in accordance with item (a) (2) (iv), which requires a report of, automatic actuation of any engineered safet (RPS)". The reactor trip was an actuation of An assessment was performed considering with the following results and conclusions: There were no operational or safety resulting in reactor trip because: There were no operational or safety resulting in reactor trip because: There were no operational or safety resulting in reactor trip because: There was stabilized in N The plant was stabilized in N The plant was already shutd temperature or pressure tran Based on the above and a review of post tri plant operated as designed, that there were and safety was assured at all times. CORRECTIVE ACTION: A. ACTION TAKEN TO RETURN A STATUS: The blown fuse in NIS IR cl	05000244	2000	- 001	- 00	6 OF 8
TEXT (If more space is required, use additional copies of NRC Form 366A)	<u> </u> (17)	<u> </u>			<u> </u>
πv	ANALVCIC OF EVENT.					
1V.	ANALISIS OF EVENI.			_		
	This event is reportable in accordance with 10 C item (a) (2) (iv) which requires a report of "Any	FR 50.73, Lic v event or con	censee E	vent Report and resulted in	System, 1 a manual	or
	automatic actuation of any engineered safety fea	ture (ESF), in	cluding	the reactor p	rotection s	ystem
	(RPS)". The reactor trip was an actuation of the	e RPS.				
	An assessment was performed considering both	the safety con	sequenc	es and impli	cations of	this event
	with the following results and conclusions:					
	There were no operational or safety cons	equences or in	mplication	ons attributed	l to the blo	wn fuse
	resulting in reactor trip because:					
	• The two reactor trip breakers ope	ned as require	xd.			
	• All control and shutdown rods the	at were withd	rawn ins	erted as desi	gned.	
	• The plant was stabilized in Mode	3.				
	• The plant was already shutdown temperature or pressure transients	with the react s related to the	or subcr e reactor	itical, so ther trip.	e were no	power,
	Based on the above and a review of post trip dat plant operated as designed, that there were no ur and safety was assured at all times.	a and past pla nreviewed saf	nt transi ety ques	ents, it can b tions, and tha	e conclude at the publi	d that the c's health
v.	CORRECTIVE ACTION:					
	A. ACTION TAKEN TO RETURN AFFEC STATUS:	CTED SYSTI	EMS TO	PRE-EVEN	T NORM/	AL.
	• The Control Room operators per Procedures E-0 and ES-0.1 and the	formed the ap he plant was s	propriat tabilized	e actions of I 1 in Mode 3.	Emergency	Operating
	• The blown fuse in NIS IR channel	el N-36 was re	eplaced.			

NRC FORM 366A (6-1998)

NRC FC	ORM 366/	A	<u> </u>		J.S. NU	ICLEAR RE	GULATORY	COMMIS	SION	
(6-1998)			T REPORT	(LER)						
		TEXT CON	ITINUATION	<u></u>						
		FACILITY NAME (1)	DOCKET (2) NUMBER (2)		LER	NUMBER (5)	PAGE	E (3)	
				YEAR	SE	QUENTIAL IUMBER	REVISION NUMBER			
R. E. (Ginna N	uclear Power Plant	05000244	2000		001 -	00	7 OF	8	
TEXT (If more sp	ace is required, use additional copies of NRC Form 366A)	<u>"</u> (17)	l				<u>u</u>		
	P									
	В.	ACTION TAKEN OK PLANNED TO P	KEVENI KE	CURRE	LINCE					
		NOTE: There are no NRC regulatory cor	nmitments in	this Lic	ensee	Event R	eport.			
		• The faulted log current amplifier	in the NIS SR	N-36 d	rawe	r was rep	laced.			
			ohonnala NT	25 and 1	N_26 -	will he -	wised to .		a '	
		 Canoration procedures for NIS in measurement of the AC ripple on 	the output of	the log	curre	nt ampli	fier.	i cquire a	ı	
		• The log current amplifier for NIS levels were acceptable.		-35 wa:	s criec	Keu Ioi I	ac uppie	. 140130		
VI.	ADD	ITIONAL INFORMATION:								
	٨									
	А.	FAILED COMPONENTS.								
		The log current amplifier is part number	2372A27G01	, manuf	actur	ed by Wo	estinghou	se Elect	ric	
		Corporation.								
	ъ	DDEVIOUS LEDS ON SIMILAR EVEN	·2TI							
	Б.	FREVIOUS LERS ON SIMILAR LVLA	10.							
		A similar LER event historical search wa	as conducted v with a similar	with the	follo ise.	wing res	ults: LER	t 90-00 3	3	
I		and LLR 1999-000 were similar events v	Willia a Dillina	1001 040						
	C	SPECIAL COMMENTS								
	с.	DI LOUID COMMIDINID.								
		None					2			
	÷									

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NRC FORM 366A (6-1998)

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	VENT REPORT	(LER)				
TEXT	CONTINUATION				·	_
FACILITY NAME (1)	DOCKET (2) NUMBER (2)		LER NUMBER (6)	PAGE	E (3
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
E. Ginna Nuclear Power Plant	05000244	2000	- 001	00	8 OF	
XT (If more space is required, use additional copies of NRC Form 3					u	=
D. IDENTIFICATION OF COMPONE	NTS REFERRED	TOIN	THIS LER:			
COMPONENT	IEEE 803	IE	EE 805			
	FUNCTI	ON SY	STEM IDEN	TIFICAT	ION	
1 (IC	1			
log current amplifier	FII	IG				
control rod	ROD	A	4			
nuclear instrument system	ЛС	IG	ſ			
main steam isolation valve	ISV	SE	3			
reactor trip breaker	52	JC	,			
	• 4					

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