

October 23, 2000

Mr. M. S. Tuckman  
Executive Vice President  
Nuclear Generation  
Duke Energy Corporation  
526 South Church Street  
P. O. Box 1006  
Charlotte, NC 28201-1006

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 RE: REQUEST FOR  
ADDITIONAL INFORMATION (TAC NOS. MA9886, MA9887, AND MA9888)

Dear Mr. Tuckman:

By letter dated August 28, 2000, you submitted a request for NRC review and approval of the methodology that will be used for the reactor coolant loop re-analysis as part of the steam generator replacement project at the Oconee Nuclear Station, Units 1, 2, and 3. In the submittal you indicated that breaks in large bore primary piping have not been considered because the NRC has approved Topical Report BAW-1847, Rev. 1. In order to complete its review, the staff requests that you provide the information shown in the enclosure regarding application of the topical report. This request has been discussed with Mr. Robert Sharpe of your staff and a target date of February 1, 2001, established to respond to this request by either supplying the information or describing a commitment to supply it.

Sincerely,

*/RA/*

David E. LaBarge, Senior Project Manager, Section 1  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270, and 50-287

Enclosure: Request for Additional Information

cc w/encl: See next page

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DOCUMENT NAME: Accession #ML003762023

OFFICE	PDII-1/PM	PDII-1/LA	PDII-1/SC
NAME	DLaBarge:mw	CHawes	REmch
DATE	10/18/00	10/18/00	10/20/00

OFFICIAL RECORD COPY

REQUEST FOR ADDITIONAL INFORMATION REGARDING  
DUKE ENERGY CORPORATION'S RE-ANALYSIS OF  
THE REACTOR COOLANT LOOP IN SUPPORT OF  
STEAM GENERATOR REPLACEMENT  
AT OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3

Per your submittal, you indicated that breaks in large bore primary piping has not been considered because the NRC has approved Topical Report BAW-1847, Rev. 1. Provide the following information to demonstrate that the analyses and results of this topical report still bounds the plant-specific applications at the Oconee Nuclear Station, Units 1, 2, and 3 after the steam generator replacement:

1. Assess the impact on applicability of BAW-1847, Rev. 1 to the Oconee Nuclear Station, Units 1, 2, and 3 due to any change of loading (dead weight, transients, OBE, and SSE) caused by the steam generator replacement.
2. Assess the impact on applicability of BAW-1847, Rev. 1 to Oconee, Units 1, 2, and 3 due to any piping material (base metal and weld) degradation and aging that might have occurred during the past 15 years of operation of the Oconee Nuclear Station, Units 1, 2, and 3. This evaluation should include the change of the pipe size (dimension) and the change of material properties such as the flow stress, Ramberg-Osgood parameters, and the fracture toughness ( $J$  or  $K_{Ic}$ ) due to degradation and aging.
3. Summarize your effort in tables similar to Table 4-11 and Table 4-12 of BAW-1847, Rev. 1. Remember that the staff is not asking you to repeat the analyses of BAW-1847, Rev. 1, but to apply your engineering judgement at every critical step to estimate the final safety factors for the primary piping of the Oconee Nuclear Station, Units 1, 2, and 3.

Oconee Nuclear Station

cc:

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