



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

October 20, 2000

Mr. Otto L. Maynard
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, KA 66839

SUBJECT: WOLF CREEK GENERATING STATION - CORRECTIONS TO AMENDMENT
NO. 123 ISSUED MARCH 31, 1999

Dear Mr. Maynard:

The Commission issued Amendment No. 123 for the Wolf Creek Generating Station (WCGS) dated March 31, 1999, that documented the conversion of the WCGS Technical Specifications (TSs) to the Improved Technical Specifications. Attached to the amendment was the Safety Evaluation dated March 31, 1999, which contained tables listing the changes to the TSs. In Table LG, "Details Relocated from the Current Technical Specifications [CTS]," the staff listed details relocated to licensee-controlled documents outside the TSs. Four corrections to Table LG were identified in your application dated November 8, 1999 (ET 99-0047) for Amendment No. 131, and in the staff's letter of December 16, 1999, approving Amendment No. 131, we issued new pages for the table with the four corrections.

Since these amendments, the staff has completed a review of your update to the Updated Safety Analysis Report (USAR) for WCGS that was submitted March 10, 2000 (ET 00-0007). In that review, the staff identified the following additional corrections to Table LG:

- (1) Change 1-16-LG, page 1 of Table LG.

A ")" was left off "(i.e., 10 CFR 50.59" in the control process column.

- (2) Change 1-52-LG, page 7 of Table LG.

Table 3.3-2 was listed under the CTS reference column and should be corrected to Table 3.3-3 because the Action 20 referenced in Table LG for this change is in Table 3.3-3.

- (3) Change 13-04-LG, page 19 of Table LG.

The word "Action" in the CTS reference column should be deleted because the detail on the ultimate heat sink being relocated from the TSs was not in the action statements.

- (4) Change 14-01-LG, page 24 of Table LG.

The change should have been deleted because the change was withdrawn from the conversion in your letter of March 5, 1999 (ET 99-0010).

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The four corrections were discussed with your staff in the conference calls of September 28 and October 6, 2000, and it was agreed that these corrections should be incorporated into Table LG of the Safety Evaluation dated March 31, 1999, for Amendment 123. Based on this, revised pages 7, 19, and 23 through 27 of Table LG for the Safety Evaluation are enclosed. The deletion of Change 14-01-LG resulted in changes to pages 23 through 27 of Table LG.

If you have any questions, please contact me at 301-415 1307 or, through the internet, at jnd@nrc.gov.

Sincerely, /RA/

Jack N. Donohew, Senior Project Manager, Section 2
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Docket No. 50-482

Enclosure: Corrected Pages to Table LG

cc w/encl: See next page

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Table LG - Details Relocated from Current Technical Specifications

Change Number (CN)	CTS Reference	Description of Relocated Details	New Location	Control Process	Characterization	Type
1-04	LG 1.7 Definition 1.7, 1.7.b, 1.7.d	Containment integrity.	ITS Bases	ITS 5.5.14	Relocation of the definition of containment integrity.	3
1-16	LG 1.23 Definition	Process controls program (PCP).	Updated Safety Analysis Report (USAR)	50.59 (i.e., 10 CFR 50.59)	Relocation of PCP definition and administrative controls description.	3
1-28	LG 1.8 Definition	Controlled leakage.	USAR	50.59	Relocation of reactor coolant pump (RCP) seal water return flow limit.	3
2-02	LG LCO 2.2.1	Reactor trip system instrumentation trip setpoints.	ITS Bases 3.3.1	ITS 5.5.14	Relocation of setpoints. The parameter used to assess operability, the allowable value, is retained in the ITS.	3
2-06	LG LCO 2.2.1, Actions	Action directions for inoperable reactor trip system instrumentation trip setpoints.	ITS Bases 3.3.1	ITS 5.5.14	The requirements in the actions are moved to ITS Table 3.3.1-1 and the explicit directions are relocated to the ITS Bases.	3
1-11	LG SR 4.0.5.a/b	Inservice Inspection (ISI) requirements.	ISI program	50.55a	The requirements duplicate 10 CFR 50.55a	4

Types of Relocated Details:

Type 1 Details of System Design.

Type 2 Descriptions of System Operation.

Type 3 Procedural Details for Meeting TS Requirements.

Type 4 Requirements Redundant to Regulations.

Type 5 Requirements Not Meeting the Safety Analyses

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Change Number (CN)	CTS Reference	Description of Relocated Details	New Location	Control Process	Characterization	Type
1-32 LG	Table 4.3-1, Functions #1, #19 and #21, and Notes 11, 16 and 18	Frequency of trip actuating device operational tests (TADOTs) for reactor trip system instrumentation.	ITS Bases	ITS 5.5.14	Relocation of the notes providing details on the TADOT and the note describing the trip for the function.	1,3
1-35 LG	3/4.3	Not Applicable to WCGS.				
1-51 LG	Table 3.3-1, Function #18	RTS interlocks.	ITS Bases	ITS 5.5.14	Relocation of the description of the P-7 inputs, i.e., P-10 and P-13.	1
1-52 LG	Table 4.3-1, Note 9 Table 3.3-1, Action 8 Table 3.3-3, Action 20	RTS instrumentation, interlocks, ESFAS interlocks, and surveillance requirements.	ITS Bases	ITS 5.5.14	Relocation of the specifics on how to verify permissive functions. The underlying requirement to verify proper permissive operation is unchanged.	3
1-57 LG	Table 3.3-1, Functions #12.a and #12.b.	Reactor coolant flow-low function in RTS instrumentation.	ITS Bases	ITS 5.5.14	The functional units are combined. Relocation of the relationships between these Functional Units and permissives P-7 and P-8.	1
2-03 LG	LCO 3.3.2 Table 3.3-4	ESFAS instrumentation trip setpoints.	ITS Bases	ITS 5.5.14	Relocation of ESFAS instrumentation trip setpoints, The allowable value, the parameter for operability, is retained in the ITS.	1

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10-26	LG	SR 4.7.4.e	CREVS surveillance.	ITS Bases	ITS 5.5.14	Relocation of the details on the method for performing the CREVS actuation surveillance.	3
10-32	LG	3/4.7	Not Applicable to WCGS				
12-06	LG	3/4.7	Not Applicable to WCGS.				
12-11	LG	3/4.7	Not Applicable to WCGS.				
13-01	LG	3/4.7	Not Applicable to WCGS.				
13-02	LG	3/4.7	Not Applicable to WCGS.				
13-04	LG	LCO 3.7.5 SR 4.7.5.b	Ultimate heat sink (UHS) operability and surveillance.	USAR	50.59	Relocation of the description of the required dam height and the requirement for the surveillance.	5
13-06	LG	3/4.7	Not Applicable to WCGS.				
13-07	LG	3/4.7	Not Applicable to WCGS.				
18-02	LG	3/4.7	Not Applicable to WCGS.				
18-04	LG	3/4.7	Not Applicable to WCGS.				
20-01	LG	3/4.7	Not Applicable to WCGS.				
1-16	LG	SR 4.8.1.1.2.f	AC sources, diesel generator (DG) start surveillance.	ITS Bases	ITS 5.5.14	Relocation of the details on the method to start the DG.	3

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Change Number (CN)	CTS Reference	Description of Relocated Details	New Location	Control Process	Characterization	Type
11-01 LG	LCO 3.9.11, Applicability and SR 4.9.11	Movement of irradiated fuel assemblies in the fuel storage areas.	USAR	50.59	Relocation of the details on when irradiated fuel is in the fuel storage areas.	3
11-04 LG	LCO 3.9.11, Action a	Water level in irradiated fuel storage pool.	USAR	50.59	Relocation of detail about restrictions on crane operation. Crane operations that could adversely affect fuel stored in the spent fuel pool are controlled as analyzed in the review of heavy load movements.	3
12-02 LG	LCO 3.9.13, Actions a and b	Emergency exhaust system, fuel building actions	USAR	50.59	Relocation of the restriction on crane operations over the spent fuel storage areas when the fuel building air cleanup system is inoperable.	5
12-09 LG	3/4.9	Not Applicable to WCGS.				
12-14 LG	SR 4.9.13.a	Emergency exhaust system - fuel building surveillance.	ITS Bases	ITS 5.5.14	Relocation of details and description of the monthly surveillances for train operability.	3
14-05 LG	SR 4.9.12	Spent fuel assembly storage surveillance.	USAR	50.59	Relocation of the requirement to keep records of the burnup analysis for all assemblies in Regions 2 or 3.	5

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Change Number (CN)	CTS Reference	Description of Relocated Details	New Location	Control Process	Characterization	Type
14-08 LG	SR 4.9.12	Spent fuel assemblies storage surveillance.	ITS Bases	ITS 5.5.14	Relocation of requirements to independently verify the burnup analysis of each spent fuel assembly prior to its storage in Regions 2 or 3 of the pool.	3
1-01 LG	3/4.11	Not Applicable to WCGS.				
1-02 LG	3/4.11	Not Applicable to WCGS.				
1-03 LG	3/4.11	Not Applicable to WCGS.				
1-01 LG	5.1.1 5.1.2 Figure 5.1-1 Figure 5.1-2	Exclusion area and low population zone.	USAR	50.59	Relocation of details in figures representing the site location/exclusion area boundary and low population zone. A text description of the site location remains in the ITS.	3
1-02 LG	5.1.3 Figure 5.1-3 Figure 5.1-4	Unrestricted areas and site boundary.	USAR	50.59	Relocation of details in map defining unrestricted areas and site boundary for radioactive gaseous and liquid effluents. The required effluent controls are retained in ITS 5.5.4.	3

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Change Number (CN)	CTS Reference	Description of Relocated Details	New Location	Control Process	Characterization	Type
2-01 LG	5.2.1 5.2.2	Containment configuration, and design pressure and temperature.	USAR	50.59	Relocation of the details on the containment design. The required controls for containment pressure and temperature are retained in ITS 3.6.4 and 3.6.5.	1
3-02 LG	5.3.2	Control rod assemblies.	USAR	50.59	Relocation of the details regarding control rod construction.	1
4-01 LG	5.4.1 5.4.2	RCS design pressure and temperature, and volume.	USAR	50.59	Relocation of the details of the RCS volume, and pressure and temperature limits. The required controls for RCS temperature and pressure are retained in ITS 3.4.1 and 3.4.2.	1
5-01 LG	5.5	Meteorological tower location.	USAR	50.59	Relocation of the meteorological tower location. Required post accident monitoring information is retained in ITS 3.3.3.	1
7-01 LG	5.7.1 Table 5.7-1	Component cyclic or transient limit.	USAR	50.59	Relocation of the table of component cyclic or transient limits. A new component cyclic or transient limit program is in ITS 5.5.5.	1

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1-04 LG	6.4	Training.	USAR	50.59	Relocation of the training functions.	5
1-04 LG	6.2.3 6.3.1.c 6.4 6.5 6.8.2 6.8.3	Independent safety engineering group (ISEG), nuclear safety review committee (NSRC), review and audit, and procedures and programs requirements.	Quality assurance plan (QAP) in USAR Chapter 17	50.54(a)	Relocation of the review and audit, the ISEG, and NSRC qualifications functions. Those requirements covered by a regulation are deleted.	4,5
1-06 LG	6.2.2 Table 6.2-1	Unit staff and shift technical advisor minimum staffing requirements.	USAR	50.59	Relocation of the details regarding minimum shift crew requirements.	4
1-07 LG	6.2.2.a	Unit staff duty shift requirements.	USAR	50.59	Revised the section to reflect the non-licensed operator staffing requirements for a single unit site to be consistent with NUREG-1431. This is not a relocation of details from the CTS.	
1-08 LG	6.2.2.e	Site fire brigade.	USAR	50.59	Relocation of the details on the fire brigade.	3
1-16 LG	6.0	Not Applicable to WCGS.				
2-04 LG	6.8.4.b	In-plant radiation monitoring.	USAR	50.59	Relocation of the in-plant monitoring program.	5
2-12 LG	6.8.4.h	Emergency diesel generator reliability program.	USAR	50.59	Relocation of the emergency diesel generator reliability program.	5

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2-13 LG	6.14.b	Offsite dose calculation manual (ODCM)	ODCM	5.5.1	Relocation of the details that the ODCM changes require review and acceptance by onsite review committees.	3
2-15 LG	6.6.1.b	Reportable event review.	USAR	50.59	Relocation of requirements for the plant review and submittal of a reportable event.	5
2-23 LG	6.0	Not Applicable to WCGS.				
3-09 LG	6.10 6.14.a	ODCM.	QAP in USAR Chapter 17	50.54(a)	Relocation of the record retention requirements and implementing procedures.	4
3-10 LG	6.11	Radiation protection program.	USAR	50.59	Relocation of the radiation protection program.	5
3-12 LG	6.13	PCP.	USAR	50.59	Relocation of the PCP.	5

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