October 16, 2000

MEMORANDUM TO: James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

FROM: Robert M. Pulsifer, Project Manager ***/RA/***
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

SUBJECT: GENERIC LETTER 96-06 CONCERNING ASSURANCE OF EQUIPMENT OPERABILITY & CONTAINMENT INTEGRITY DURING DESIGN BASIS ACCIDENT CONDITIONS (TAC NO. M96833/34)

The enclosed draft request for additional information (RAI) was transmitted by facsimile on October 6, 2000, to Paul Russell of Northeast Nuclear Energy Company (licensee). Review of the RAI would allow the licensee to determine and agree upon a schedule to respond to the RAI. This memorandum and the enclosure do not convey a formal request for information or represent an NRC staff position.

Docket No. 50-336 and 423

Enclosures:. Draft Request for Additional Information

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Docket No. 50-336 and 443

Enclosures:. Draft Request for Additional Information

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DRAFT REQUEST FOR ADDITIONAL INFORMATION
BY THE OFFICE OF NUCLEAR REACTOR REGULATION
MILLSTONE NUCLEAR POWER STATION, UNITS 2 AND 3
GENERIC LETTER 96-06
NORTHEAST NUCLEAR ENERGY COMPANY
DOCKET NO. 50-336 AND 50-423

Millstone 2

In the 1-28-97 submittal, the licensee states that 9 penetrations are to be reviewed and corrective actions implemented prior to startup. Provide a description of how the thermal pressurization concern has been addressed for each of these penetrations.

RELAP5 is a computer code with a largely empirical basis for its closure relations. Therefore, RELAP5 must be assessed against experimental data that is applicable to the present analysis. Please provide the RELAP5 assessment that was performed that qualifies it for the present application. Describe how the range of conditions in the experiments correspond to the water hammer calculations.

The RELAP5 output is used in structural load calculations. Provide the assessment of the overall load methodology against applicable experimental data that qualify it for the present application. Describe how the range of conditions in the experiments correspond to the water hammer calculations.

Also, in your response to Question 7 of the submittal, you indicated that the "procedural change would be made to instruct operators to delay restarting an idle RBCCW pump (restarting of the pump will be based on the existing post-accident containment conditions)." The decision to start the pump should be based on the worst conditions that were experienced in containment, not the existing conditions.

Millstone 3

In the 5-11-98 submittal, the licensee describes two cases where thermal pressurization can occur. One case is where piping inside containment could pressurize and damage inboard containment isolation valves (CIVs). Provide a description of how damage of the inboard CIVs is prevented for each penetration. Another case is where isolated vent and drain lines may be thermally pressurized. The licensee provides a discussion of an assessment based on use of Appendix F of the ASME Code, but indicates that this may not be the long-term approach. The staff has determined that an analysis in accordance with the provisions of Appendix F is one acceptable method of addressing the thermal pressurization issue. Provide a description of the long-term solution for each of the vent and drain lines.

Enclosure