

OCT 06 2000  
LRN-00-0396  
LCR S99-13



United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Gentlemen:

**SUPPLEMENTAL INFORMATION FOR  
REQUEST FOR AMENDMENT  
REACTOR COOLANT SYSTEM – SAFETY VALVES AND  
PLANT SYSTEMS – TURBINE CYCLE – SAFETY VALVES  
SALEM GENERATING STATION  
UNIT NOS. 1 AND 2  
DOCKET NOS. 50-272 AND 50-311**

In accordance with 10CFR50.90, on September 26, 2000, PSEG Nuclear LLC submitted Salem License Change Request (LCR) S99-13 (ref. letter LRN-00-0317) requesting a revision to the Technical Specifications (TS) for the Salem Generating Station Units No 1 and 2. The changes proposed in that submittal consist of revisions to the Reactor Coolant System – Safety Valves, and the Plant Systems – Turbine Cycle – Safety Valves Technical Specifications. Subsequent to the submission of Salem License Change Request S99-13, an item was identified in that submittal requiring correction.

The copies of the Salem Unit 1 and Unit 2 Bases for TS 3/4.7.1.1, Turbine Cycle –Safety Valves provided in the original submittal of LCR S99-13 did not include a complete mark-up of the proposed change. The correct Salem Unit 1 and Unit 2 TS Bases 3/4.7.1.1 mark-up pages are provided in Attachment 1. The corrections to the marked-up pages provided in Attachment 1 are necessary for the TS Bases to reflect the requested change to TS 3/4.7.1.1 to remove the limits to the Power Range Neutron Flux High setpoint imposed during operation with three reactor coolant loops with inoperable Main Steam Safety Valves. PSEG Nuclear LLC has concluded that the changes to the marked-up pages provided in Attachment 1 do not alter the conclusions reached in the 10CFR50.92 No Significant Hazards analysis submitted with LCR S99-13. The removal of references

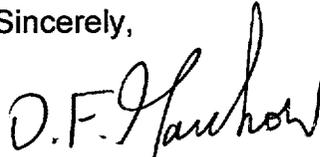
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OCT 06 2000

to requirements for plant operation with three reactor coolant loops contained in the TS 3/4.7.1.1, and its Bases, is administrative in nature, and is being performed to bring TS 3/4.7.1.1 into agreement with changes made to the TS under the Salem Margin Recovery Program that removed TS requirements for plant operation with three reactor coolant loops.

Should you have any questions regarding this request, please contact Brooke Knieriem, Salem Licensing, at (856) 339-1782.

Sincerely,



D. F. Garchow

Vice President – Technical Support

/rbk  
Affidavit  
Attachment

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Attachment 1

**Marked-up Pages**

**Salem Unit 1 and Unit 2 Bases 3/4.7.1.1, Turbine Cycle – Safety Valves**

**Salem Unit 1 – Page B3/4 7-1**

**Salem Unit 2 – Page B3/4 7-1**

### 3/4.7 PLANT SYSTEMS

#### BASES

#### 3/4.7.1 TURBINE CYCLE

##### 3/4.7.1.1 SAFETY VALVES

Insert I

The OPERABILITY of the main steam line code safety valves ensures that the secondary system pressure will be limited to within 110% of its design pressure of 1085 psig during the most severe anticipated system operational transient. The maximum relieving capacity is associated with a turbine trip from 100% RATED THERMAL POWER coincident with an assumed loss of condenser heat sink (i.e., no steam bypass to the condenser).

The specified valve lift settings and relieving capacities are in accordance with the requirements of Section III of the ASME Boiler and Pressure Code, 1971 Edition. The total relieving capacity for all valves on all of the steam lines is 16,655,268 lbs/hr which is 115 percent of the total secondary steam flow of 14,459,360 lbs/hr at 100% RATED THERMAL POWER. A minimum of 2 OPERABLE safety valves per OPERABLE steam generator ensures that sufficient relieving capacity is available for the allowable THERMAL POWER restriction in Table 3.7-2.

STARTUP and/or POWER OPERATION is allowable with safety valves inoperable within the limitations of the ACTION requirements on the basis of the reduction in secondary system steam flow and THERMAL POWER required by the reduced reactor trip settings of the Power Range Neutron Flux channels. The reactor trip setpoint reductions are derived on the following bases:

For 4 loop operation

$$SP = \frac{(X) - (Y)(V)}{X} \times (109)$$

For 3 loop operation

$$SP = \frac{(X) - (Y)(U)}{X} \times (76)$$

Where:

SP = reduced reactor trip setpoint in percent of RATED THERMAL POWER

V = maximum number of inoperable safety valves per steam line

Replace  
with  
Insert J

### 3/4.7 PLANT SYSTEMS

#### BASES

#### 3/4.7.1 TURBINE CYCLE

##### 3/4.7.1.1 SAFETY VALVES

Insert I

The OPERABILITY of the main steam line code safety valves ensures that the secondary system pressure will be limited to within 110% of its design pressure of 1085 psig during the most severe anticipated system operational transient. The maximum relieving capacity is associated with a turbine trip from 100% RATED THERMAL POWER coincident with an assumed loss of condenser heat sink (i.e., no steam bypass to the condenser).

The specified valve lift settings and relieving capacities are in accordance with the requirements of Section III of the ASME Boiler and Pressure Code, 1971 Edition. The total relieving capacity for all valves on all of the steam lines is 16,655,268 lbs/hr which is 115% of the total secondary steam flow of 14,459,360 lbs/hr at 100% RATED THERMAL POWER. A minimum of 2 OPERABLE safety valves per OPERABLE steam generator ensures that sufficient relieving capacity is available for the allowable THERMAL POWER restriction in Table 3.7-2.

STARTUP and/or POWER OPERATION is allowable with safety valves inoperable within the limitations of the ACTION requirements on the basis of the reduction in secondary system steam flow and THERMAL POWER required by the reduced reactor trip settings of the Power Range Neutron Flux channels. The reactor trip setpoint reductions are derived on the following bases:

For 4 loop operation

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For 3 loop operation

$$SP = \frac{(X) - (Y)(U)}{X} \times (76)$$

Where:

SP = reduced reactor trip setpoint in percent of RATED THERMAL POWER

V = maximum number of inoperable safety valves per steam line

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