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October 5, 2000

Re: Indian Point Unit No. 2
Docket No. 50-247
NL-00-123

Document Control Desk
US Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555-0001

Subject: Correction of Typographical Errors in the Technical Specifications

Consolidated Edison Company of New York, Inc. (Con Edison) has determined that there are two typographical errors in the Indian Point Unit No. 2 Technical Specifications.

The first typographical error, addition of the word "approved," appeared in License Amendment 209 (Reference 1) on page 4.13-3. The Con Edison letter that provided the retyped version of page 4.13-3 was Reference 2. Prior to the issuance of Amendment 209, page 4.13-3 had last been revised in Amendment 201 (Reference 3). It was in this amendment that the word "approved" was removed from Specification 4.13.A.2.e. However, in developing Reference 1, the word "approved" was inadvertently used. Please find attached the corrected page 4.13-3 for Amendment 209.

The second typographical error, in which Footnote *4 used the value "350°F" instead of "381°F," appeared in License Amendment 187 (Reference 4) in Table 4.1-1 on page 7 of 7. The Con Edison letter that provided the retyped version of Table 4.1-1 on page 7 of 7 was Reference 5. Prior to the issuance of Amendment 187, this page had last been revised in Amendment 184 (Reference 6) in which Table 4.1-1 was reformatted (with the correct value of 381°F in Footnote *4). Prior to Amendment 184, the unnumbered footnote appeared on page 4 of 7 of Table 4.1-1. It was in a prior change to this page, which took place in Amendment 179 (Reference 7), that the value of "350°F" was replaced with "381°F." However, in developing Reference 5, the value of "350°F" was inadvertently used. Currently, Footnote *4 appears in Table 4.1-1 on page 8 of 8, which was updated in Amendment 205 (Reference 8). Please find attached the corrected Table 4.1-1, page 8 of 8 for Amendment 205. Finally, it is important to note, that while Footnote *4 had the wrong temperature value of 350°F, plant procedures still reflected the correct value of 381°F.

Therefore, based on the above and in accordance with SECY-96-238 (Reference 9), Con Edison hereby requests that the attached pages be used as the corrected pages, respectively, for Specification 4.13.A.2.e, page 4.13-3 (Amendment 209), and Table 4.1-1, page 8 of 8 (Amendment 205).

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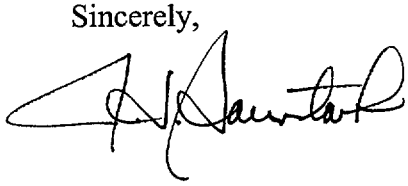
In accordance with 10 CFR 50.91, a copy of this letter is being submitted to the designated New York State official.

There are no commitments contained in this correspondence.

Should you or your staff have any questions regarding this submittal, please contact Mr. John F. McCann, Manager, Nuclear Safety and Licensing.

Sincerely,



References and cc: Next pages

Attachment

References:

1) USNRC Letter to A. A. Blind from J. F. Harold, "Indian Point Nuclear Generating Unit No. 2 - Re: Issuance of Amendment Regarding Steam Generator Inspection Requirements (TAC No. MA8488)," dated April 28, 2000.

2) Con Edison Letter to the USNRC Document Control Desk from A. A. Blind, "Proposed Technical Specification Amendment Consisting of Administrative Changes to Steam Generator Inspection Requirements," dated March 17, 2000.

3) USNRC Letter to A. A. Blind from J. F. Harold, "Issuance of Amendment for Indian Point Nuclear Generating Unit No. 2 Allowing a One-Time Extension of the Steam Generator Inspection Interval (TAC No. MA4526)," dated June 9, 1999.

4) USNRC Letter to S. E. Quinn from G. F. Wunder, "Issuance of Amendment for Indian Point Nuclear Generating Unit No. 2 (TAC No. M90164)," dated October 30, 1996.

5) Con Edison Letter to the USNRC Document Control Desk from S. E. Quinn, "Proposed Changes In Technical Specification Surveillance Intervals To Accommodate A 24 Month Fuel Cycle," dated August 1, 1996.

6) USNRC Letter to S. E. Quinn from G. F. Wunder, "Issuance of Amendment for Indian Point Nuclear Generating Unit No. 2 (TAC No. M89039)," dated October 12, 1995.

7) USNRC Letter to S. E. Quinn from G. F. Wunder, "Issuance of Amendment for Indian Point Nuclear Generating Unit No. 2 (TAC No. M89039)," dated December 20, 1994.

8) USNRC Letter to A. A. Blind from J. F. Harold, "Indian Point Nuclear Generating Unit No. 2 - Issuance Of Exigent Amendment Re: Allowing One-Time Extension of Surveillance Intervals (TAC No. MA5302)," dated October 29, 1999.

9) USNRC Policy Issue SECY-96-238, "Guidance for Correction of Technical Specification Typographical Errors," dated November 19, 1996 (Note: As stated in SECY-96-238, approved by Negative Consent).

cc: Mr. Hubert J. Miller
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ATTACHMENT

CORRECTED PAGES

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.
INDIAN POINT UNIT NO. 2
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OCTOBER 2000

c. Unscheduled steam generator examinations shall be required in the event there is a primary to secondary leak exceeding technical specifications, a seismic occurrence greater than an operating basis earthquake, a loss-of-coolant accident requiring actuation of engineered safeguards, or a major steamline or feedwater line break.

d. Unscheduled examinations may include only the steam generator(s) affected by the leak or other occurrence.

e. In case of an unscheduled steam generator examination, the profilometry tensile strain criterion shall be the same as contained in the program for the last scheduled steam generator inspection.

3. Basic Sample Selection and Examination

a. At least 12% of the tubes in each steam generator to be examined shall be subjected to a hot-leg examination.

b. At least 25% of the tubes inspected in Specification 4.13.A.3.a above shall be subjected to a cold-leg examination.

c. At least 20% of a random sample of tubes containing sleeves shall be subjected to an examination throughout the sleeved portion of the tube.

d. Tubes selected for examination shall include, but not be limited to, tubes in areas of the tube bundle in which degradation has been reported, either at Indian Point 2 in prior examinations, or at other utilities with similar steam generators.

e. Examination for deformation ("dents") shall be either by eddy current or by profilometry.

f. Examination for degradation other than deformation shall be by eddy current techniques. A 700-mil diameter probe shall be used unless previous data indicates that a 700-mil diameter probe would not pass through the tube. If the 700-mil diameter probe cannot pass through the tube, the largest size probe that is expected to pass through the tube shall be used. In all cases a probe with at least a 610-mil diameter shall be used, except for the examination of the U-bends and the cold-legs of tubes in rows 2 through 5. For these examinations, a 540-mil diameter probe may be used, provided it is justified by profilometry measurement within the tensile strain criterion.

Table 4.1-1

Minimum Frequencies for Checks, Calibrations and Tests of Instrument Channels

Footnotes:

*1 By means of the movable incore detector system.

*2 Prior to each reactor startup if not done previous week.

*3 Monthly visual inspection of condensate weirs only.

*4 Within 31 days prior to entering a condition in which the Control Rod Protection System is required to be operable unless the reactor trip breakers are manually opened during RCS cooldown prior to T_{cold} decreasing below 381°F and the breakers are maintained opened during RCS cooldown when T_{cold} is less than 381°F.

*5 Except when block valve operator is deenergized.

*6 Within 31 days prior to entering a condition in which OPS is required to be operable and at monthly intervals thereafter when OPS is required to be operable.

*7 Acceptable criteria for calibration are provided in Table II.F-13 of NUREG-0737.

*8 Calibration will be performed using calibration span gas.

*9 Each train shall be tested at least every 62 days on a staggered test basis (i.e., one train per month).