

November 2, 2000

Mr. S. E. Scace - Director
Nuclear Oversight and Regulatory Affairs
c/o Mr. David A. Smith
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385-0128

SUBJECT: MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3 - ISSUANCE OF
AMENDMENT RE: IMPLEMENTATION OF 10 CFR PART 50, APPENDIX J,
OPTION B (TAC NO. MA8745)

Dear Mr. Scace:

The Commission has issued the enclosed Amendment No. 186 to Facility Operating License No. NPF-49 for the Millstone Nuclear Power Station, Unit No. 3, in response to your application dated April 19, 2000, as supplemented on August 31, 2000.

The amendment implements a performance-based Containment Leakage Testing Program in accordance with 10 CFR Part 50, Appendix J, Option B as a substitute for the requirements of 10 CFR Part 50, Appendix J, Option A. The use of this option requires the implementation of a program based on Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," and modification of the Technical Specifications to reflect this program. The amendment modifies the following Technical Specifications (TS) Sections: Definition 1.7, "CONTAINMENT INTEGRITY"; Sections 3/4.6.1.1, "Containment Systems, Primary Containment, CONTAINMENT INTEGRITY"; 3/4.6.1.2, "Containment Systems, Primary Containment, Containment Leakage"; 3/4.6.1.3, "Containment Systems, Primary Containment, Containment Air Locks"; 3/4.6.1.6, "Containment Systems, Primary Containment, Containment Structural Integrity"; 3/4.6.6.3, "Containment Systems, Secondary Containment Structural Integrity"; and 6.8, "Procedures and Programs." The Bases for these TSs are modified to address the proposed changes.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Victor Nerses, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosures: 1. Amendment No. 186 to NPF-49
2. Safety Evaluation

cc w/encls: See next page

Mr. S. E. Scace - Director
Nuclear Oversight and Regulatory Affairs
c/o Mr. David A. Smith
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385-0128

November 2, 2000

SUBJECT: MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3 - ISSUANCE OF
AMENDMENT RE: IMPLEMENTATION OF 10 CFR PART 50, APPENDIX J,
OPTION B (TAC NO. MA8745)

Dear Mr. Scace:

The Commission has issued the enclosed Amendment No. 186 to Facility Operating License No. NPF-49 for the Millstone Nuclear Power Station, Unit No. 3, in response to your application dated April 19, 2000, as supplemented on August 31, 2000.

The amendment implements a performance-based Containment Leakage Testing Program in accordance with 10 CFR Part 50, Appendix J, Option B as a substitute for the requirements of 10 CFR Part 50, Appendix J, Option A. The use of this option requires the implementation of a program based on Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," and modification of the Technical Specifications to reflect this program. The amendment modifies the following Technical Specifications (TS) Sections: Definition 1.7, "CONTAINMENT INTEGRITY"; Sections 3/4.6.1.1, "Containment Systems, Primary Containment, CONTAINMENT INTEGRITY"; 3/4.6.1.2, "Containment Systems, Primary Containment, Containment Leakage"; 3/4.6.1.3, "Containment Systems, Primary Containment, Containment Air Locks"; 3/4.6.1.6, "Containment Systems, Primary Containment, Containment Structural Integrity"; 3/4.6.6.3, "Containment Systems, Secondary Containment Structural Integrity"; and 6.8, "Procedures and Programs." The Bases for these TSs are modified to address the proposed changes.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Victor Nerses, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosures: 1. Amendment No. 186 to NPF-49
2. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION:

PUBLIC	OGC	PDI-2 R/F	RSummers, RI
EAdensam	JLinville, RI	TClark	ACRS
GHill (2)	WBeckner	JClifford	

Accession Number: ML003758538

*See previous concurrence

OFFICE	PDI-2/PM	PDI-2/LA	PDI-2/PM	SPLB*	OGC	PDI-2/SC
NAME	VNerses	TLClark	AWang	RL for GHubbard	CMarco	JClifford
DATE	11/1/00	11/1/00	11/1/00	9/22/00	10/20/00	10/31/00

OFFICIAL RECORD COPY

Millstone Nuclear Power Station
Unit 3

cc:

Ms. L. M. Cuoco
Senior Nuclear Counsel
Northeast Utilities Service Company
P. O. Box 270
Hartford, CT 06141-0270

Edward L. Wilds, Jr., Ph.D.
Director, Division of Radiation
Department of Environmental Protection
79 Elm Street
Hartford, CT 06106-5127

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

First Selectmen
Town of Waterford
15 Rope Ferry Road
Waterford, CT 06385

Mr. M. H. Brothers
Vice President - Nuclear Operations
Northeast Nuclear Energy Company
P.O. Box 128
Waterford, CT 06385

Mr. M. R. Scully, Executive Director
Connecticut Municipal Electric
Energy Cooperative
30 Stott Avenue
Norwich, CT 06360

Mr. J. T. Carlin
Vice President - Human Services - Nuclear
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385

Mr. F. C. Rothen
Vice President - Nuclear Work Services
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385

Ms. Cynthia Arcate, Vice President
Generation Investments
New England Power Company
25 Research Drive
Westborough, MA 01582

Mr. Allan Johanson, Assistant Director
Office of Policy and Management
Policy Development & Planning Division
450 Capitol Avenue - MS# 52ERN
P. O. Box 341441
Hartford, CT 06134-1441

Mr. R. P. Necci
Vice President - Nuclear Technical Services
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385

Millstone Nuclear Power Station
Unit 3

cc:

Mr. John W. Beck, President
Little Harbor Consultants, Inc.
44 Nichols Road
Cohasset, MA 02025-1166

Mr. L. J. Olivier
Senior Vice President and
Chief Nuclear Officer - Millstone
Northeast Nuclear Energy Company
P.O. Box 128
Waterford, CT 06385

Mr. C. J. Schwarz
Station Director
Northeast Nuclear Energy Company
P.O. Box 128
Waterford, CT 06385

Senior Resident Inspector
Millstone Nuclear Power Station
c/o U.S. Nuclear Regulatory Commission
P. O. Box 513
Niantic, CT 06357

Nicholas J. Scobbo, Jr., Esquire
Ferriter, Scobbo, Caruso, & Rodophele, P.C.
75 State Street, 7th Floor
Boston, MA 02108-1807

Mr. G. D. Hicks
Director - Nuclear Training Services
Northeast Nuclear Energy Company
P.O. Box 128
Waterford, CT 06385

Mr. William D. Meinert
Nuclear Engineer
Massachusetts Municipal Wholesale
Electric Company
P.O. Box 426
Ludlow, MA 01056

Mr. B. D. Kenyon
President and Chief Executive Officer-
NNECO
Northeast Nuclear Energy Company
P.O. Box 270
Hartford, CT 06141-0270

Mr. D. A. Smith
Manager - Regulatory Affairs
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385

Ms. Nancy Burton
147 Cross Highway
Redding Ridge, CT 00870

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 186
License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated April 19, 2000, as supplemented on August 31, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.186 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: November 2, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 186

FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following pages of the Appendix A Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

1-2

3/4.6-1

3/4.6-2

3/4.6-3

3/4.6-5

3/4.6-6

3/4.6-10

3/4.6-23

6-17

B 3/4.6-1

B 3/4.6-1a

B 3/4.6-2

Insert

1-2

3/4.6-1

3/4.6-2

3/4.6-3

3/4.6-5

3/4.6-6

3/4.6-10

3/4.6-23

6-17

6-17a

B 3/4.6-1

B 3/4.6-1a

B 3/4.6-2

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 186

TO FACILITY OPERATING LICENSE NO. NPF-49

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

DOCKET NO. 50-423

1.0 INTRODUCTION

On September 12, 1995, the U.S. Nuclear Regulatory Commission (NRC) approved issuance of a revision to 10 CFR Part 50, Appendix J, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors" which was subsequently published in the *Federal Register* on September 26, 1995, and became effective on October 26, 1995. The NRC added Option B, "Performance-Based Requirements," to allow licensees to voluntarily replace the prescriptive testing requirements of 10 CFR Part 50, Appendix J, with testing requirements based on both overall performance and the performance of individual components.

By letter dated April 19, 2000, as supplemented on August 31, 2000, the Northeast Nuclear Energy Company, et al. (NNECO/the licensee), submitted a request for changes to the Millstone Nuclear Power Station, Unit No. 3 Technical Specifications (TSs). The requested changes would implement a performance-based Containment Leakage Testing Program in accordance with 10 CFR Part 50, Appendix J, Option B as a substitute for the requirements of 10 CFR Part 50, Appendix J, Option A. The use of this option requires the implementation of a program based on Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," and modification of the TSs to reflect this program. The amendment will modify the following TSs Sections: Definition 1.7, "CONTAINMENT INTEGRITY"; Sections 3/4.6.1.1, "Containment Systems, Primary Containment, CONTAINMENT INTEGRITY"; 3/4.6.1.2, "Containment Systems, Primary Containment, Containment Leakage"; 3/4.6.1.3, "Containment Systems, Primary Containment, Containment Air Locks"; 3/4.6.1.6, "Containment Systems, Primary Containment, Containment Structural Integrity"; 3/4.6.6.3, "Containment Systems, Secondary Containment Structural Integrity"; and 6.8, "Procedures and Programs." The Bases for these TSs will be modified to address the proposed changes. The August 31, 2000, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

2.0 BACKGROUND

Compliance with 10 CFR Part 50, Appendix J, provides assurance that the primary containment, including those systems and components which penetrate the primary containment, do not exceed the allowable leakage rate specified in the TSs and Bases. The allowable leakage rate is determined so that the leakage rate assumed in the safety analyses is not exceeded.

On February 4, 1992, the NRC published a notice in the *Federal Register* (57 FR 4166) discussing a planned initiative to begin eliminating requirements marginal to safety which impose a significant regulatory burden. Title 10 of the *Code of Federal Regulations*, Part 50, Appendix J, "Primary Containment Leakage Testing for Water-Cooled Power Reactors," was considered for this initiative and the staff undertook a study of possible changes to this regulation. The study examined the previous performance history of domestic containments and examined the effect on risk of a revision to the requirements of Appendix J. The results of this study are reported in NUREG-1493, "Performance-Based Leak-Test Program".

Based on the results of this study, the staff developed a performance-based approach to containment leakage rate testing. On September 12, 1995, the NRC approved issuance of this revision to 10 CFR Part 50, Appendix J, which was subsequently published in the *Federal Register* on September 26, 1995, and became effective on October 26, 1995. The revision added Option B, "Performance-Based Requirements," to Appendix J to allow licensees to voluntarily replace the prescriptive testing requirements of Appendix J with testing requirements based on both overall and individual component leakage rate performance.

Regulatory Guide 1.163, "Performance-Based Containment Leak Test Program," dated September 1995, was developed as a method acceptable to the NRC staff for implementing Option B. This regulatory guide states that the Nuclear Energy Institute (NEI) guidance document NEI 94-01, Rev. 0, "Industry Guideline for Implementing Performance-Based Option of 10 CFR Part 50, Appendix J," provides methods acceptable to the NRC staff for complying with Option B with four exceptions which are described therein.

Option B requires that Regulatory Guide 1.163 or another implementation document used by a licensee to develop a performance-based leakage testing program must be included, by general reference, in the plant TSs. The licensee has referenced Regulatory Guide 1.163 in the proposed Millstone TSs.

Regulatory Guide 1.163 specifies an extension in Type A test frequency to at least one test in 10 years based upon two consecutive successful tests. Type B tests may be extended up to a maximum interval of 10 years based upon completion of two consecutive successful tests and Type C tests may be extended up to 5 years based on two consecutive successful tests.

By letter dated October 20, 1995, NEI proposed TSs to implement Option B. After some discussion, the staff and NEI agreed on final TSs which were transmitted to NEI in a letter dated November 2, 1995. These TSs are to serve as a model for licensees to develop plant-specific TS in preparing amendment requests to implement Option B.

In order for a licensee to determine the performance of each component, factors that are indicative of or affect performance, such as an administrative leakage limit, must be

established. The administrative limit is selected to be indicative of the potential onset of component degradation. Although these limits are subject to NRC inspection to assure that they are selected in a reasonable manner, they are not TS requirements. Failure to meet an administrative limit requires the licensee to return to the minimum value of the test interval.

Option B requires that the licensee maintain records to show that the criteria for Type A, B, and C tests have been met. In addition, the licensee must maintain comparisons of the performance of the overall containment system and the individual components to show that the test intervals are adequate. These records are subject to NRC inspection.

3.0 EVALUATION

The licensee's April 19, 2000, letter to the NRC, as supplemented on August 31, 2000, proposes to establish a "Primary Containment Leakage Rate Testing Program" and proposes to add this program to the TSs. The program references Regulatory Guide 1.163, "Performance-Based Containment Leak Test Program," dated September 1995, which specifies methods acceptable to the NRC for complying with Option B. The licensee proposes changes to existing TS Sections: Definition 1.7, "CONTAINMENT INTEGRITY"; Sections 3/4.6.1.1, "Containment Systems, Primary Containment, CONTAINMENT INTEGRITY"; 3/4.6.1.2, "Containment Systems, Primary Containment, Containment Leakage"; 3/4.6.1.3, "Containment Systems, Primary Containment, Containment Air Locks"; 3/4.6.1.6, "Containment Systems, Primary Containment, Containment Structural Integrity"; 3/4.6.6.3, "Containment Systems, Secondary Containment Structural Integrity"; and 6.8, "Procedures and Programs." The corresponding Bases were also modified.

Option B permits a licensee to choose Type A; or Type B and C; or Type A, B, and C; testing to be done on a performance basis. The licensee has elected to perform Type A, B, and C testing on a performance basis.

The TS changes proposed by the licensee are in compliance with the requirements of Option B and are consistent with the guidance of Regulatory Guide 1.163. Further, the proposed TS changes generally conform to the model TS guidance provided in the NRC letter to NEI dated November 2, 1995, despite the different format of the licensee's current TSs. The specific TS changes are discussed below in 3.1.

Additionally, the licensee has proposed a related TS change to Appendix J that goes beyond the scope of the conversion to Option B; this is discussed in Section 3.2 below.

3.1 Option B TS Changes

Current TS Definition 1.7.d is being revised by deleting the reference to Specification 3.6.1.2 and replacing it with reference to the Containment Leakage Rate Testing Program. This change is in compliance with the requirements of Option B of 10 CFR 50, Appendix J and consistent with the guidance of RG 1.163, September 1995, and the model TS of our letter dated November 2, 1995, and, therefore, is acceptable.

Current Surveillance Requirement (SR) 4.6.1.1.c is being revised to delete the specific details of the Type B testing. These details are included in the Containment Leakage Rate Testing Program. This change is in compliance with the requirements of Option B of 10 CFR 50,

Appendix J and consistent with the guidance of RG 1.163, September 1995, and the model TS of our letter dated November 2, 1995, and, therefore, is acceptable.

Current TS 3.6.1.2 is being revised to delete explicit containment leakage rate acceptance criteria and replace it with a reference to the Containment Leakage Rate Testing Program. This change is in compliance with the requirements of Option B of 10 CFR 50, Appendix J and consistent with the guidance of RG 1.163, September 1995, and the model TS of our letter dated November 2, 1995, and, therefore, is acceptable.

Current SR 4.6.1.2 is being revised by deleting all test requirements from SR 4.6.1.2 and replacing them with a reference to the Containment Leakage Rate Testing Program. The Containment Leakage Rate Testing Program has been defined in Section 6.8.4 of the TSs. This program contains the necessary acceptance criteria and test frequencies associated with Type A, B, and C tests. The provisions of Specification 4.0.2 remain "not applicable" to the testing and that specific statement is included in Section 6.8.4. This change is in compliance with the requirements of Option B of 10 CFR 50, Appendix J and consistent with the guidance of RG 1.163, September 1995, and the model TS of our letter dated November 2, 1995, and, therefore, is acceptable.

Current TS Section 3.6.1.3.b is being revised to delete explicit air lock leakage rate acceptance criteria and replace them with a reference to the Containment Leakage Rate Testing Program. This change is in compliance with the requirements of Option B of 10 CFR 50, Appendix J and consistent with the guidance of RG 1.163, September 1995, and the model TS of our letter dated November 2, 1995, and, therefore, is acceptable.

Current SRs 4.6.1.3.a and b are being revised to delete the air lock testing requirements and acceptance criteria and replace them with a reference to the Containment Leakage Rate Testing Program. Also, the footnotes on page 3/4 6-6 have been deleted. These footnotes are being maintained in TS Section 6.8.4. This change is in compliance with the requirements of Option B of 10 CFR 50, Appendix J and consistent with the guidance of RG 1.163, September 1995, and the model TS of our letter dated November 2, 1995, and, therefore, is acceptable.

Current SRs 4.6.1.6.1 and 4.6.6.3 are being revised to delete the specific reference to Type A test frequency and replace it with "at the frequency specified in the Containment Leakage Rate Testing Program." This change is in compliance with the requirements of Option B of 10 CFR 50, Appendix J and consistent with the guidance of RG 1.163, September 1995, and the model TS of our letter dated November 2, 1995, and, therefore, is acceptable.

New Section 6.8.4.f, "Containment Leakage Rate Testing Program" will describe the Containment Leakage Rate Testing Program and leakage criteria. By letter dated August 31, 2000, NNECO revised this proposed TS by deleting the wording 'NEI 94-01, "Industry Guideline for Performance-Based Option of 10 CFR 50, Appendix J," Revision 0; ANSI/ANS-56.8-1994, "Containment Leakage Testing Requirements;" as modified by approved exceptions.' This modification will make the wording of TS 6.8.4.f consistent with the wording provided in Technical Specification Task Force change traveler TSTF-52, Rev. 3, which the staff approved on March 8, 2000. The new Section 6.8.4.f, "Containment Leakage Testing Program," includes an additional requirement that is not part of the current TSs. This requirement is item 2) b, which addresses air lock testing acceptance criterion of each air lock door. The added acceptance criterion is consistent with the acceptance criterion in NUREG-1431 for the testing

of each air lock door. The staff has reviewed these sections and the additional requirement and finds that they are consistent with Regulatory Guide 1.163 and NEI 94-01, and are, therefore, acceptable.

A statement confirming the applicability of Specification 4.0.3 in the case of the Containment Leakage Rate Testing Program is added to TS Section 6.8.4.f. In addition, as noted earlier, two notes from SR 4.6.1.2 have been added. These changes are administrative in nature and are proposed for the purpose of clarity and, therefore, are acceptable.

3.2 Related TS Changes

TS Section 3.6.1.2

The wording in the current action statement of Specification 3.6.1.2 is being replaced with "With the containment leakage rates exceeding the limits, restore the leakage rates to within limits within 1 hour or be in at least HOT STANDBY within the next 6 hours and COLD SHUTDOWN within the following 30 hours." The current action statement does not specify the needed action if the required containment leakage rates are not met and the plant is above 200 °F. In this case, Specification 3.0.3 would apply. The proposed timeframes are consistent with the requirement in Specification 3.0.3. The proposed change provides clearer guidance for the action statement and is consistent with the current requirements and NUREG-1431. The staff finds the proposed TS to be essentially equivalent to the existing TS and therefore, finds it acceptable.

3.3 Bases Section Changes

Bases Sections 3/4.6.1.2, "Containment Leakage"; 3/4.6.1.3, "Containment Air Locks"; 3/4.6.1.6, "Containment Structural Integrity"; and 3/4.6.1.7, "Containment Ventilation System" are being revised to reflect the above changes, including a reference to the Containment Leakage Rate Testing Program. The staff does not object to the proposed TS Bases changes.

3.4 Summary

In summary, the staff has reviewed the changes to the TSs and associated Bases proposed by the licensee, for Option B implementation, and finds that they are in compliance with the requirements of Appendix J, Option B, and are consistent with the guidance of Regulatory Guide 1.163, and are, therefore, acceptable. Further, the staff finds the additional change discussed in section 3.2 above to be acceptable on the bases discussed therein.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes

surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (65 FR 51359). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A. Wang

Date: November 2, 2000