

October 5, 2000

Mr. Oliver D. Kingsley, President  
Nuclear Generation Group  
Commonwealth Edison Company  
Executive Towers West III  
1400 Opus Place, Suite 500  
Downers Grove, IL 60515

SUBJECT: LASALLE - ISSUANCE OF AMENDMENTS (TAC NOS. MA8825 AND MA8827)

Dear Mr. Kingsley:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 141 to Facility Operating License No. NPF-11 and Amendment No. 127 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2, respectively. The amendments are in response to your application dated April 25, 2000.

The amendments revise Technical Specification 3/4.9.5, "Communications" to allow the movement of a control rod in a fueled core cell in Operational Condition 5 to be exempt from the requirement that direct communication be maintained between the control room and the refueling platform personnel when the rod is moved with its normal drive system.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Donna M. Skay, Project Manager, Section 2  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-373, 50-374

- Enclosures: 1. Amendment No. 141 to NPF-11
- 2. Amendment No. 127 to NPF-18
- 3. Safety Evaluation

cc w/encls: See next page

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\*see previous page for concurrence

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

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Office of Nuclear Reactor Regulation

Docket Nos. 50-373, 50-374

Enclosures: 1. Amendment No. 141 to NPF-11  
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3. Safety Evaluation

cc w/encls: See next page

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Units 1 and 2

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- 2 -

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Units 1 and 2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

October 5, 2000

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 141  
License No. NPF-11

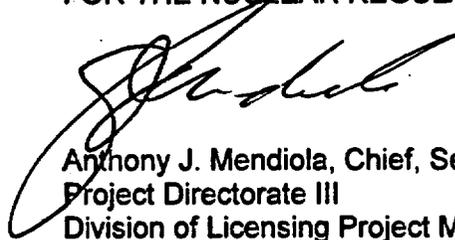
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated April 25, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) **Technical Specifications and Environmental Protection Plan**

The Technical Specifications contained in Appendix A, as revised through Amendment No. 141 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Anthony J. Mendiola, Chief, Section 2  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 5, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 141

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains a vertical line indicating the area of change.

REMOVE

3/4 9-7

INSERT

3/4 9-7

## REFUELING OPERATIONS

### 3/4.9.5 COMMUNICATIONS

#### LIMITING CONDITION FOR OPERATION

---

3.9.5 Direct communication shall be maintained between the control room and refueling platform personnel.

APPLICABILITY: OPERATIONAL CONDITION 5, during CORE ALTERATIONS\*.

ACTION:

When direct communication between the control room and refueling platform personnel cannot be maintained, immediately suspend CORE ALTERATIONS.

#### SURVEILLANCE REQUIREMENTS

---

4.9.5 Direct communication between the control room and refueling platform personnel shall be demonstrated within one hour prior to the start of and at least once per 12 hours during CORE ALTERATIONS.

---

\*Except movement of control rods with their normal drive system.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 127  
License No. NPF-18

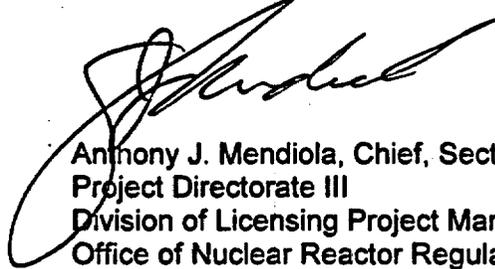
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated April 25, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 127, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Anthony J. Mendiola, Chief, Section 2  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 5, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 127

FACILITY OPERATING LICENSE NO. NPF-18

DOCKET NO. 50-374

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains a vertical line indicating the area of change.

REMOVE

3/4 9-7

INSERT

3/4 9-7

## REFUELING OPERATIONS

### 3/4.9.5 COMMUNICATIONS

#### LIMITING CONDITION FOR OPERATION

---

3.9.5 Direct communication shall be maintained between the control room and refueling platform personnel.

APPLICABILITY: OPERATIONAL CONDITION 5, during CORE ALTERATIONS\*.

ACTION:

When direct communication between the control room and refueling platform personnel cannot be maintained, immediately suspend CORE ALTERATIONS.

#### SURVEILLANCE REQUIREMENTS

---

4.9.5 Direct communication between the control room and refueling platform personnel shall be demonstrated within one hour prior to the start of and at least once per 12 hours during CORE ALTERATIONS.

---

\*Except movement of control rods with their normal drive system.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 141 TO FACILITY OPERATING LICENSE NO. NPF-11  
AND AMENDMENT NO. 127 TO FACILITY OPERATING LICENSE NO. NPF-18  
COMMONWEALTH EDISON COMPANY  
LASALLE COUNTY STATION, UNITS 1 AND 2  
DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

By letter dated April 25, 2000, Commonwealth Edison Company (ComEd, the licensee) requested changes to the Technical Specifications (TS) for LaSalle County Station, Units 1 and 2. The proposed change would add a footnote to TS 3/4.9.5, "Communications," that would allow the movement of a control rod without maintaining direct communication between the control room and the refueling platform personnel when the control rod is moved with its normal drive system. A similar footnote was included in the original TS and its deletion was requested by the licensee by letter dated August 13, 1999, and approved by the NRC by Amendment Nos. 136 and 121 to the facility operating licenses for LaSalle, Units 1 and 2, respectively. Subsequent to issuance of the amendments, the licensee discovered that removal of the entire footnote resulted in a more restrictive procedure for exercising control rods in Operational Condition 5 than is intended by TS 3/4.9.5. Therefore, the licensee requested that the portion of the footnote related to movement of control rods be re-inserted.

2.0 EVALUATION

TS 3.9.5 requires that direct communication be maintained between the control room and refueling platform personnel during CORE ALTERATIONS. Amendment Nos. 136 and 121 revised the definition of CORE ALTERATIONS to exclude the movement of instrumentation and control rods if there are no fuel assemblies in the associated cell. The amendment also removed several footnotes that were no longer necessary under the revised definition. One of the footnotes removed was associated with TS 3.9.5 which had allowed the movement of incore instrumentation and control rods with their normal drive system to be excluded from the communication requirements. The licensee subsequently determined that the removal of this footnote results in an unnecessary burden when exercising control rods since it requires that refueling platform be staffed whenever a control is exercised from the control room in Operational Condition 5. This requirement was not intended by the licensee's August 13, 1999,

submittal or the staff's safety evaluation dated October 18, 1999, in response to that application.

TS 3/4.9.3, "Control Rod Position," allows the movement of one control rod at a time, in a fueled core cell, under the control of the reactor mode switch Refuel position one-rod-out interlock. As a result of removing the footnote to TS 3.9.5, the licensee was required to staff the refueling platform whenever control rods were exercised in accordance with TS 3.9.3.

Therefore, the licensee proposes to reinsert the portion of the footnote that allows movement of control rods with their normal drive system to be excluded from the communication requirements of TS 3.9.5.

The licensee stated in its April 25, 2000, application that sufficient controls are currently required by the TS that ensure that the control rod being moved under the one-rod-out interlock would automatically insert if a reactivity excursion was to occur. These include TS Table 3.3.1-1, "Reactor Protection System Instrumentation," which requires the following functional units to be operable in Operational Condition 5 with any control rod withdrawn:

- 1.a. "Intermediate Range Monitors: Neutron Flux - High"
- 1.b. "Intermediate Range Monitors: Inoperative"
8. "Scram Discharge Volume Water Level - High"
11. "Reactor Mode Switch Shutdown Position," and
12. "Manual Scram"

The purpose of TS 3.9.5, as stated in the TS Bases, is to ensure that "refueling station personnel can be promptly informed of significant changes in the facility status or core reactivity during movement of fuel within the reactor pressure vessel." Therefore, because the proposed TS continues to require that communications be maintained during fuel movement and only excludes the requirement for control rod movements, the proposed change meets the intent of the TSs as stated in the Bases.

The staff has determined that the proposed change meets the intent of the TS as stated in the Bases. In addition, adequate controls exist in the TS to ensure that the control rod being moved will insert if a reactivity excursion were to occur. Therefore, the staff finds the proposed change to TS 3/4.9.5 acceptable.

### **3.0 STATE CONSULTATION**

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

### **4.0 ENVIRONMENTAL CONSIDERATION**

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is

no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (65 FR 37422). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### **4.0 CONCLUSION**

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Skay

Date: October 5, 2000