Nuclear Operating Company
South Texas Project Electric Generating Station P.O. Box 289 Wadsworth, Texas 77483

September 28, 2000
NOC-AE-00000923
File No.: G20.02.02
G21.02.02
10CFR50.59
STI: 31168920

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

South Texas Project
Units 1 and 2
Docket Nos. STN 50-498, STN 50-499
Technical Specification Bases Change

The South Texas Project Technical Specification Bases Section SR 4.8.1.1.2.e.11 has been changed pursuant to 10CFR50.59. Words were added to the section to identify the sequencer as a support system for the associated diesel generator and those components actuated by a Mode I signal. Attached is a copy of the revised Technical Specification Bases page.

If there are any questions, please contact S. M. Head at (361) 972-7136 or me at (361) 972-7206.

Mark A. McBurnett
Director,
Quality & Licensing

kaw

Attachment: Revised Technical Specification Bases Page 3/4 8-13 (1 page)

A001

cc:

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U. S. Nuclear Regulatory Commission
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A.C. SOURCES, D.C. SOURCES, and ONSITE POWER DISTRIBUTION (Continued)

SR 4.8.1.1.2.e.11

As required by Regulatory Guide 1.108, paragraph 2.a.(2), each DG is required to demonstrate proper operation for the DBA loading sequence to ensure that voltage and frequency are maintained within the required limits. Under accident conditions, prior to connecting the DGs to their respective busses, all loads are shed except load center feeders and those motor control centers that power Class 1E loads (referred to as "permanently connected" loads). Upon reaching 90% of rated voltage and frequency, the DGs are then connected to their respective busses.

Loads are then sequentially connected to the bus by the automatic load sequencer. This sequencing logic controls the permissive and starting signals to motor breakers to prevent overloading of the DGs due to high motor starting currents. The 10% load sequence time interval tolerance ensures that sufficient time exists for the DG to restore frequency and voltage prior to applying the next load and that safety analysis assumptions regarding ESF equipment time delays are not violated.

The sequencer is considered a support system for the associated diesel generator and those components actuated by a Mode I signal (CR 00-10707).

The Frequency of 18 months is consistent with the recommendation of Regulatory Guide 1.108, paragraph 2.a.(2), takes into consideration unit conditions required to perform the Surveillance, and is intended to be consistent with expected fuel cycle lengths.

SR 4.8.1.1.2.e.12

This SR verifies that the diesel will not start when the emergency stop lockout feature is tripped. This prevents any further damage to the diesel engine or generator.

SR 4.8.1.1.2.e.13

This SR verifies the requirements of Branch Technical Position PSB-1 that the load shedding scheme automatically prevents load shedding during the sequencing of the emergency loads to the bus. It also verifies the reinstatement of the load shedding feature upon completion of the load sequencing action.

SR 4.8.1.1.2.f

This Surveillance demonstrates that the DG starting independence has not been compromised. Also, this Surveillance demonstrates that each engine can achieve proper speed within the specified time when the DGs are started simultaneously.

Duke Energy

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Vice President

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September 26, 2000

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: McGuire Nuclear Station
Docket Nos. 50-369, 50-370
Proposed Correction to License Amendment Nos. 195/176, Facility Operating Licenses NPF-9 and NPF-17, (TAC Nos. MA8696 and MA8697)

By letter dated September 22, 2000, the NRC issued approved License Amendment No. 195 to Facility Operating License NPF-9 and Amendment No. 176 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. The amendments revised the Technical Specifications and associated Bases to reference the Westinghouse Best Estimate Large Break Loss-of-Coolant Accident (LOCA) analysis methodology described in WCAP-12945-P-A, March 1998.

By letter dated September 22, 1999, the NRC issued approved License Amendment No. 188 to Facility Operating License NPF-9 and Amendment No. 169 to Facility Operating License NPF-17. The amendments revised various sections of the Technical Specifications to permit use of Westinghouse's Robust Fuel Assemblies for future core reloads.

During the implementation review of the approved changes, discrepancies were identified on page 5.6-4 of the Technical Specifications. It was discovered that changes previously issued in License Amendment 188/169 had been erroneously eliminated through an administrative error.

Attachment 1 contains a markup of page 5.6-4 from Amendment 188/169 and adds the reference to WCAP-12945-P-A from Amendment 195/176. Attachment 2 contains the corrected page containing both approvals to be implemented during Unit 2, Cycle 14.

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Questions regarding this submittal should be directed to Kay Crane, McGuire Regulatory Compliance at (704) 875-4306.

HB Barron

H. B. Barron, Vice President
McGuire Nuclear Station

cc: Mr. Frank Rinaldi, Project Manager
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Mr. Scott Shaeffer
Senior Resident Inspector
McGuire Nuclear Station

Attachment 1

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

4. DPC-NE-2011PA, "Duke Power Company Nuclear Design Methodology for Core Operating Limits of Westinghouse Reactors," March, 1990 (DPC Proprietary).

5. DPC-NE-3001PA, "Multidimensional Reactor Transients and Safety Analysis Physics Parameter Methodology," November, 1991 (DPC Proprietary).

6. DPC-NF-2010A, "Duke Power Company McGuire Nuclear Station Catawba Nuclear Station Nuclear Physics Methodology for Reload Design," June, 1985.

7. DPC-NE-3002A, Rev. 3 "FSAR Chapter 15 System Transient Analysis Methodology," SER dated February 5, 1999.

8. DPC-NE-3000PA, Rev. 2 "Thermal-Hydraulic Transient Analysis Methodology," SER dated October 14, 1998. (DPC Proprietary).

9. DPC-NE-1004A, Rev. 1, "Nuclear Design Methodology Using CASMO-3/SIMULATE-3P," SER dated April 26, 1996.

10. DPC-NE-2004P-A, Rev. 1, "Duke Power Company McGuire and Catawba Nuclear Stations Core Thermal-Hydraulic Methodology using VIPRE-01," SER dated February 20, 1997 (DPC Proprietary).

11. DPC-NE-2005P-A, Rev. 1, "Thermal Hydraulic Statistical Core Design Methodology," SER dated November 7, 1996 (DPC Proprietary).

12. DPC-NE-2008P-A, "Fuel Mechanical Reload Analysis Methodology Using TACO3," SER dated April 3, 1995 (DPC Proprietary).

13. WCAP-10054-P-A, "Westinghouse Small Break ECCS Evaluation Model using the NOTRUMP Code," August 1985 (W Proprietary).

14. DPC-NE-2009-P-A, "Westinghouse Fuel Transition Report, "SER dated September 22, 1999 (DPC Proprietary).

15. WCAP-12945-P-A, Volume 1 (Revision 2) and Volumes 2-5 (Revision 1), "Code Qualification Document for Best-Estimate LOSS of Coolant Analysis," March 1998, (W Proprietary).

(continued)

Attachment 2

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

4. DPC-NE-2011PA, "Duke Power Company Nuclear Design Methodology for Core Operating Limits of Westinghouse Reactors," March, 1990 (DPC Proprietary).

5. DPC-NE-3001PA, "Multidimensional Reactor Transients and Safety Analysis Physics Parameter Methodology," November, 1991 (DPC Proprietary).

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14. DPC-NE-2009-P-A, "Westinghouse Fuel Transition Report, "SER dated September 22, 1999 (DPC Proprietary).

15. WCAP-12945-P-A, Volume 1 (Revision 2) and Volumes 2-5 (Revision 1), "Code Qualification Document for Best-Estimate Loss of Coolant Analysis," March 1998, (W Proprietary).

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