



A PECO Energy/British Energy Company

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2130-00-20257
September 26, 2000

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

Subject: Oyster Creek Nuclear Generating Station (OCNGS)
Docket No. 50-219
Facility License No. DPR-16
Re: Technical Specification Change Request No. 278
Response to Request for Additional Information

Reference: Correspondence No. 1940-00-20153 dated June 30, 2000, "Technical Specification Change Request No. 278"

The referenced license amendment request confirms the applicability of the Safety Limit Minimum Critical Power Ratio (SLMCPR) for the next operating cycle (Cycle 18). There is no change to the value of the SLMCPR.

The NRC staff verbally requested additional information concerning the referenced license amendment request on September 18, 2000. A telephone conference was conducted on September 20, 2000 to discuss AmerGen's proposed responses to the request. AmerGen's response to the request is attached and reflects the clarifications discussed during the telephone conference.

ADDI

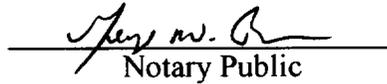
If additional information is required, please contact Paul F. Czaya of Oyster Creek Licensing at 609-971-4139.

Very truly yours,



Ron J. DeGregorio
Vice President
Oyster Creek

Sworn to and subscribed before me this 26th day of September, 2000.



Notary Public

GEORGE W. BUSCH
NOTARY PUBLIC OF NEW JERSEY
My Commission Expires Aug. 8, 2006

- c: Administrator, USNRC Region I
USNRC Oyster Creek Senior Project Manager
USNRC Oyster Creek Senior Resident Inspector
Kent Tosch, Director, Bureau of Nuclear Engineering, Department of Environmental
Protection, State of New Jersey
The Honorable William J. Boehm, Mayor of Lacey Township

**Response to Request for Additional Information
Re: Technical Specification Change Request No. 278**

NRC Question

1. Provide the number of bundles for each fuel type in the Cycle 18 core and cycle for which they were introduced into the core.

AmerGen Response

There are two fuel types to be used in the Cycle 18 core. They are:

1. GE9B-P8DWB348-12GZ-80M-145, which is GE9 fuel of 3.48 wt. % U_{235} .
2. GE9B-P8DWB338-11GZ-80M-145, which is GE9 fuel of 3.38 wt. % U_{235} .

There will be 432 assemblies of the first type and 128 of the second type. The table below shows the cycle assemblies of each type were introduced into the reactor:

Fuel Type and Batch Size of the Cycle 18 Core

Fuel Type	# of Assemblies (fresh)	# of Assemblies (Cycle 17)	# of Assemblies (Cycle 16)	# of Assemblies (Cycle 15)
GE9 (3.48 wt. % U_{235})	136	144	148	4
GE9 (3.38 wt. % U_{235})	48	40	40	0

It should be noted that if an adjustment is needed to the makeup of the Cycle 18 core as a result of fuel inspections or other circumstances, then the replacement fuel assemblies would come from those assemblies intended for discharge. This results in the addition of less reactivity into the core and core design parameters would remain bounding. There would be no effect on the calculated value of the Safety Limit Minimum Critical Power Ratio for Cycle 18.

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NRC Question

2. Provide an explanation that a generic calculation is still shown in Table 1 of the Attachment even though approved methodologies are given in Reference 1.

AmerGen Response

The information provided in Table 1 of Technical Specification Change Request No. 278 regarding the generic analysis was, as stated, for comparison purposes. A cycle-specific analysis was performed using approved methodologies for Oyster Creek Cycle 18. The license amendment request, however, did not propose to eliminate NRC reviews of SLMCPR in the future. This is an apparent oversight and will be addressed in a future license amendment request.